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A LINEAR PROGRAMMING REACTOR

REFUELING MODEL

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B.S., Kansas State University, 1970

A MASTER'S THESIS

submitted in partial fulfillment of the

requirements for the degree

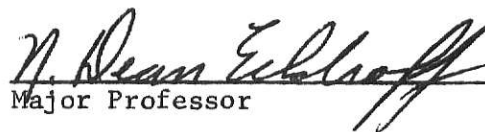
MASTER OF SCIENCE

Department of Nuclear Engineering

KANSAS STATE UNIVERSITY
Manhattan, Kansas

1971

Approved by:


Major Professor

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NOMENCLATURE

a_{ij}	coefficient of variable j in equation i
b_i	right hand side coefficient
BFS	basic feasible solution
c_j	coefficient of variable j in the objective function
En_{max}	maximum enrichment
En_{min}	minimum enrichment
f	thermal utilization
I_m	amount of nuclide m
I_m^0	initial amount of nuclide m
I_{im}	amount of nuclide m in irradiation period i
k_{eff}	effective multiplication factor
k_{∞}	infinite multiplication factor
OF	objective function
p	resonance escape probability
P	non-leakage probability
t	time
U_{im}	amount of nuclide m added to or removed from the reactor after irradiation period i
V	volume of the reactor core
Z	objective function

Greek symbols

ϵ	fast fission factor
η_0	average number of neutrons liberated directly by fission for every thermal neutron absorbed in the fuel at the beginning of each irradiation period

η_f	average number of neutrons liberated by fission for every thermal neutron absorbed in the fuel at the end of each irradiation period
η_m	ratio of the neutrons produced by fission to the number of neutrons absorbed in fissile species m
ν_m	ratio of the number of neutrons produced by fission to the number of neutrons absorbed in fission by fissile species m
σ_{am}	microscopic absorption cross section for nuclide m
σ_{cm}	microscopic capture cross section for nuclide m
σ_{fm}	microscopic fission cross section for nuclide m
Σ_s	macroscopic cross section of reactor structural material
$\overline{\phi}(t)$	average neutron flux
θ	neutron fluence

Subscript (m) for symbols

0	Pu-240
1	Pu-241
5	U-235
6	U-236
7	fission product pairs
8	U-238
95	Pu-239 from thermal neutron absorption in U-235
98	Pu-239 from thermal neutron absorption in U-238
N7	Np-237
P8	Pu-238

1.0 INTRODUCTION

As the number of nuclear power plants increases, nuclear fuel management considerations take on greater urgency. The dollar magnitude of fuel in the nuclear picture was illustrated by Benedict (1): A 1000 Mwe pressurized water reactor (PWR) or boiling water reactor (BWR), over 30 years at 65 per cent capacity will spend 303 million dollars on fuel. This illustrates the importance of the optimization of fuel usage; an error of one per cent could lead to a loss of 3.03 million dollars for the electric utility.

Several techniques have been used for fuel optimization but most deal with the maximization of fuel burnup. Two such techniques appear in works by Fagan (2) and Tabak (3). These techniques are not best suited to a utility because the utility must schedule refueling shut down times according to their seasonal load curve and other system capacity. Hence, the utility must refuel during a proper refueling window. This is a predicted time of the year, usually in the spring or fall, when power demands are low enough that the utility can shut down a reactor to refuel and still maintain necessary power requirements with the operation of other generating units of the system. Thus, scheduling of refueling times is a very important consideration.

It is the purpose of this work to develop a reactor model to obtain the most economical use of nuclear fuel for the utility over a period of five refueling periods. The reactor was assumed to be a base load reactor, operating at a steady power, and refueling was assumed to take place once a year during the predicted refueling window. A linear programming model was developed to fulfill this objective.

2.0 CONCEPTS OF LINEAR PROGRAMMING

2.1 Definition of Linear Programming

Linear programming (LP) deals with those problems in which all relations among the variables are linear. This includes the objective function (OF), that function which is to be maximized or minimized, and all problem constraints. LP is a technique which can be used to optimize the OF, which must be a linear function of a parameter such as cost, profit, or products produced. The constraints developed for the model must be linear combinations of such things as the amount of time, money, man power, or raw materials available.

"The general linear programming problem can be described as follows: Given a set of m linear inequalities or equations in r variables, we wish to find non-negative values of these variables which will satisfy the constraints and maximize or minimize some linear function of the variables (4)."

Mathematically this can be written as, given

$$a_{i1}x_1 + a_{i2}x_2 + \dots + a_{ir}x_r \{ \leq = \geq \} b_i, \quad i = 1, \dots, m, \quad (2.1.1)$$

where b_i and a_{ij} , $i=1, \dots, m$ and $j=1, \dots, r$, are known constants. The optimal values of x_j will satisfy Eq. (2.1.1) with

$$x_j \geq 0, \quad j = 1, \dots, r, \quad (2.1.2)$$

and maximize or minimize a linear objective function,

$$Z = c_1x_1 + \dots + c_rx_r, \quad (2.1.3)$$

where c_j , $j=1, \dots, r$, are known constants.

2.2 Linear Programming Example

As an illustration of a simple LP problem consider a fuel fabricator who produces a type A and B fuel element. The company has three shops X, Y, Z in which the fuel elements are produced. Figure 2.2.1 shows: 1) the hours required in each shop per unit of each element; 2) the total available shop time per month; 3) the profit on the sale of one unit of any one of the elements. The company wishes to know the optimum production schedule for maximizing profit when the contribution to profit is 2000 dollars for a type A fuel element and 1000 dollars for a type B fuel element.

The OF selected in this problem is profit:

$$Z = 2000x_1 + 1000x_2 , \quad (2.2.1)$$

where x_1 is the number of type A elements produced and x_2 is the number of type B elements produced.

This OF is subject to the following constraints:

$$2x_1 + 3x_2 \leq 600 , \quad (2.2.2)$$

$$1x_1 + \frac{10}{3}x_2 \leq 500 , \quad (2.2.3)$$

$$3x_1 + x_2 \leq 600 , \quad (2.2.4)$$

Since we cannot produce negative quantities, i.e., to have a feasible solution to Eqs. (2.2.2 - 2.2.4), the following are additional implicit restrictions of the model:

$$x_1, x_2 \geq 0 . \quad (2.2.5)$$

In tableau form (4) the model is given as in Figure 2.2.2.

The constraints and OF (dotted line) are shown in Fig. 2.2.3. The shaded area is the feasibility polygon, i.e., convex space, which contains any point which will satisfy the constraints of Eqs. (2.2.2 - 2.2.5). Because the OF

has a constant slope, it is maximized by passing through the extreme point D, $x_1 = 171.43$ and $x_2 = 85.71$. The optimal value of the OF at this point is $Z = \$428,571$, i.e.,

$$Z = (\$2000) (171.43) + (\$1000) (85.71) = \$428,571 . \quad (2.2.6)$$

This example shows the geometrical interpretation of a simple linear programming problem and that the set of feasible solutions described by Eqs. (2.2.2 - 2.2.5) formed a convex region. It was also shown that at least one extreme point of the convex region was optimal. When a problem involves more than three dimensions the solution becomes hard to visualize and nearly impossible to show graphically. The number of basic feasible solutions (BFS), extreme points, can soon become large (but finite) and the problem of examining these BFS's for the optimal solution may take an inordinate amount of time. Thus the simplex and revised simplex methods of finding the optimal solution were developed (2). The procedure used in each of these is a series of systematic steps from one initial BFS to other BFS's and finally, in a finite number of steps, to an optimal BFS, if one exists. The optimal BFS is found in such a way that the value of the OF at each step, for a maximization problem, is greater than the preceding step.

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Shop	Fuel Elements		Total hours available per month
	A	B	
X	3	3	600
Y	1	10/3	500
Z	2	1	600
unit profit	\$2000	\$1000	

Figure 2.2.1. The data for the example.

	C_j	2000	1000	0	0	0	
C_B	Vectors in basis	a_1	a_2	a_3	a_4	a_5	b
0	a_3	2	3	1	0	0	600
0	a_4	1	10/3	0	1	0	500
0	a_5	3	1	0	0	1	600
	$Z_j - C_j$	-2000	-1000	0	0	0	0

Figure 2.2.2. The initial tableau for the example.

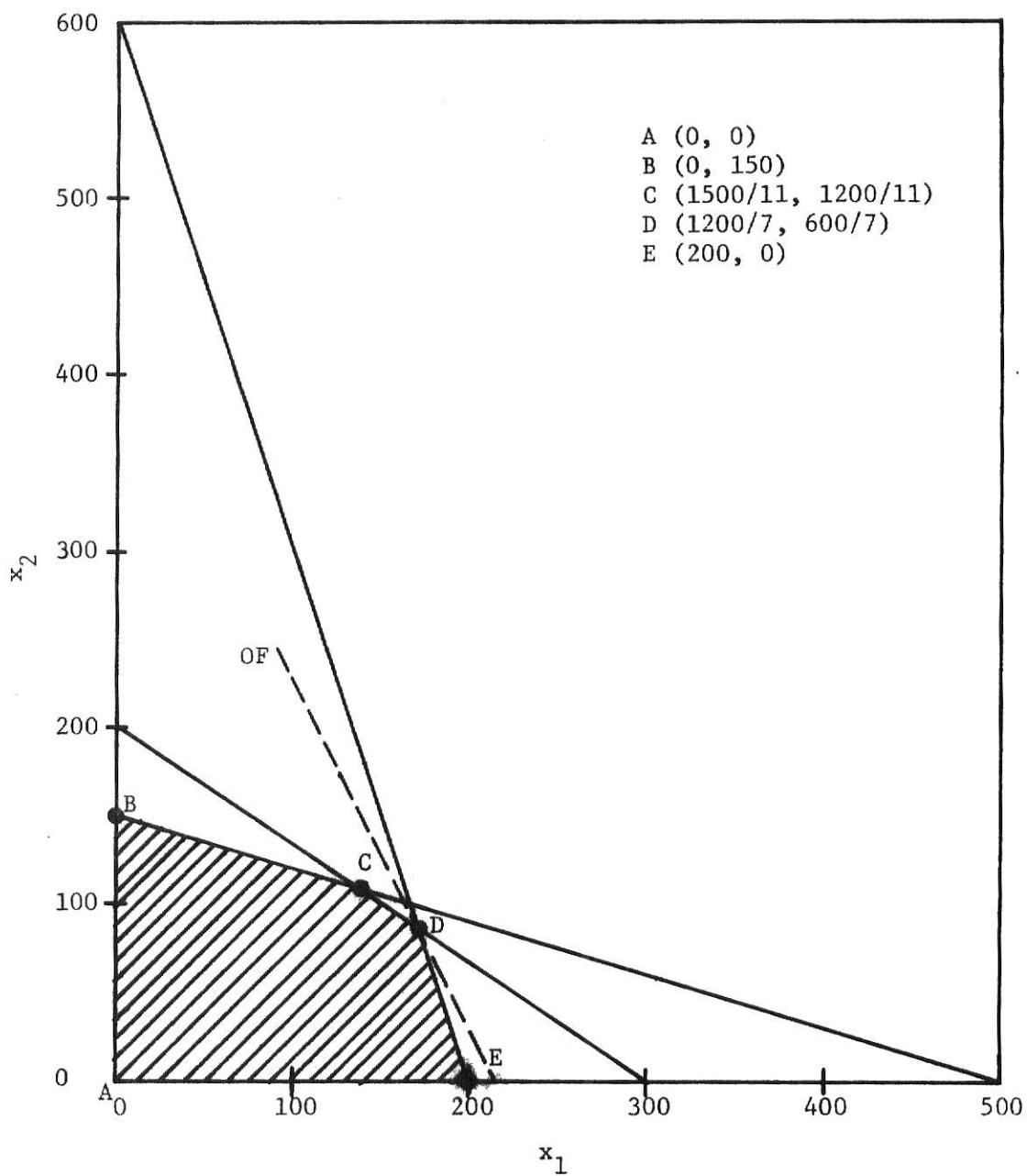


Figure 2.2.3 Graphical Solution to Example Problem.

3.0 MODEL DEVELOPMENT

3.1 The Reactor

Because of the limited supply of uranium much research is being done on fast breeder and plutonium recycle reactors. Due to difficult technical problems the fast breeder reactor does not seem to be an economically feasible reactor until at least 1985. But, the plutonium recycle reactor should be ready for the utility industry by 1975.

Because of this, a plutonium recycle reactor was chosen for this work. The effects of plutonium recycle were studied to determine its advantages or disadvantages to U-235 minimization during refueling.

3.2 Nuclide Production and/or Burnup

The reactor was assumed to be loaded initially with only U-235 and U-238. During operation of the reactor the production of the following nuclides was considered; U-236, Pu-238, Pu-239, Pu-240, Pu-241, Np-237, and fission products. The following assumptions were used in the development of a computer burnup code: 1) One homogeneous region, 2) a known neutron fluence for each operation period, 3) one neutron energy group (thermal), 4) no U-238 fission, 5) negligible decay of nuclides, and 6) batch irradiation.

With the above assumptions, nuclide concentrations during continuous irradiation can be expressed by a set of first order, linear, ordinary differential equations with constant coefficients. The independent variable in these equations is neutron fluence, θ , i.e.,

$$\theta = \int_0^t \bar{\phi}(t') dt' . \quad (3.2.1)$$

$\bar{\phi}(t)$ is the average neutron flux at which the reactor operates over the irradiation period.

The concentration of U-235, I_5 , during irradiation is

$$\frac{dI_5}{d\theta} = -I_5 \sigma_{a5} . \quad (3.2.2)$$

The solution to Eq. (3.2.2) is

$$I_5 = K_{11} I_5^0, \quad (3.2.3)$$

where

$$K_{11} = \exp(-\sigma_{a5} \theta). \quad (3.2.4)$$

The concentration of U-238, I_8 , is expressed by

$$\frac{dI_8}{d\theta} = -I_8 \sigma_{a8}, \quad (3.2.5)$$

which integrates to

$$I_8 = K_{33} I_8^0, \quad (3.2.6)$$

where

$$K_{33} = \exp(-\sigma_{a8}\theta). \quad (3.2.7)$$

For build up of U-236, I_6 , the differential equation is

$$\frac{dI_6}{d\theta} = I_5 \sigma_{c5} - I_6 \sigma_{a6}. \quad (3.2.8)$$

The solution to Eq. (3.2.8) is

$$I_6 = K_{21} I_5^0 + K_{22} I_6^0, \quad (3.2.9)$$

where

$$K_{21} = \frac{\sigma_{c5}}{A} (B - C), \quad (3.2.10)$$

$$K_{22} = C, \quad (3.2.11)$$

$$A = \sigma_{a6} - \sigma_{a5}, \quad (3.2.12)$$

$$B = \exp(-\sigma_{a5}\theta), \quad (3.2.13)$$

$$C = \exp(-\sigma_{a6}\theta). \quad (3.2.14)$$

The production of Pu-239 was assumed to come from two reactions, the thermal absorption of neutrons in U-238 and U-235. The differential equation for production of Pu-239 from thermal absorption in U-238 is

$$\frac{dI_{98}}{d\theta} = I_8 \sigma_{a8} + \epsilon p_1 (1 - p) (I_5 \sigma_{a5} \eta_5 + I_{98} \sigma_{a9} \eta_9) - I_{98} \sigma_{a9}. \quad (3.2.15)$$

The first term of Eq. (3.2.15) is the thermal absorption of a neutron in U-238 to form U-239 which then decays by two betas to form Pu-239. The second term represents the resonance absorption of fast neutrons produced by U-235 and Pu-239 fission. The last term is the removal of Pu-239 due to thermal absorption.

The solution to Eq. (3.2.15) is

$$I_{98} = K_{41} I_5^0 + K_{43} I_8^0 + K_{44} I_{98}^0, \quad (3.2.16)$$

where

$$K_{41} = \frac{C5}{D} (E - B), \quad (3.2.17)$$

$$K_{43} = CB(1 - E), \quad (3.2.18)$$

$$K_{44} = E, \quad (3.2.19)$$

$$E = \exp(C9\theta), \quad (3.2.20)$$

$$D = \sigma_{a5} + C9, \quad (3.2.21)$$

$$C5 = \epsilon P_1 (1 - p) \eta_5 \sigma_{a5}, \quad (3.2.22)$$

$$C8 = -\sigma_{c8}/C9, \quad (3.2.23)$$

$$C9 = \epsilon P_1 (1 - p) \eta_9 \sigma_{a9} - \sigma_{a9}. \quad (3.2.24)$$

Before considering the production of Pu-239 from thermal absorption in U-235, Np-237 and Pu-238 must be considered. The concentration of Np-237, I_{N7} , can be found from

$$\frac{dI_{N7}}{d\theta} = \sigma_{c6} I_6 - \sigma_{cN7} I_{N7}. \quad (3.2.25)$$

The solution to this is

$$I_{N7} = K_{91} I_5^0 + K_{92} I_6^0 + K_{97} I_{N7}^0, \quad (3.2.26)$$

where

$$K_{91} = \frac{\sigma_{c6} \sigma_{c5}}{A} \left[\frac{B - F}{G} + \frac{F - C}{H} \right], \quad (3.2.27)$$

$$K_{92} = \frac{\sigma_{c6}}{G} (C - F), \quad (3.2.28)$$

$$K_{97} = F, \quad (3.2.29)$$

$$F = \exp(-\sigma_{cN7}), \quad (3.2.30)$$

$$G = \sigma_{cN7} - \sigma_{a5}, \quad (3.2.31)$$

$$H = \sigma_{cN7} - \sigma_{a6}, \quad (3.2.32)$$

Then for Pu-238, I_{P8} , the differential equation is

$$\frac{dI_{P8}}{d\theta} = \sigma_{cN7}I_{N7} - \sigma_{cP8}I_{P8}, \quad (3.2.33)$$

which integrates to

$$I_{P8} = K_{81}I_5^0 + K_{82}I_6^0 + K_{87}I_{N7}^0 + K_{88}I_{P8}^0, \quad (3.2.34)$$

where

$$K_{81} = \frac{\sigma_{cN7}\sigma_{c6}\sigma_{c5}}{A} \left[\frac{B - J}{LG} - \frac{C - J}{MH} \right] + \frac{\sigma_{cN7}\sigma_{c6}\sigma_{c5}}{NA} \left[\frac{1}{H} - \frac{1}{G} \right] (F - J), \quad (3.2.35)$$

$$K_{82} = \frac{\sigma_{cN7}\sigma_{c6}}{H} \left[\frac{C - J}{M} - \frac{F - J}{N} \right], \quad (3.2.36)$$

$$K_{87} = \frac{\sigma_{cN7}}{N} (F - J), \quad (3.2.37)$$

$$K_{88} = J, \quad (3.2.38)$$

$$J = \exp(-\sigma_{cP8}\theta), \quad (3.2.39)$$

$$L = \sigma_{cP8} - \sigma_{a5}, \quad (3.2.40)$$

$$M = \sigma_{cP8} - \sigma_{a6}, \quad (3.2.41)$$

$$N = \sigma_{cP8} - \sigma_{cN7}. \quad (3.2.42)$$

For Pu-239 produced from thermal fission in U-235, I_{95} , the differential equation is

$$\frac{dI_{95}}{d\theta} = \sigma_{cP8}I_{P8} - \sigma_{a9}I_{95}, \quad (3.2.43)$$

which has the solution:

$$I_{95} = K_{101}I_5^0 + K_{102}I_6^0 + K_{107}I_{N7}^0 + K_{108}I_{P8}^0 + K_{109}I_{95}^0, \quad (3.2.44)$$

where

$$K_{101} = \frac{\sigma_{cP8} \sigma_{cN7} \sigma_{c6} \sigma_{c5}}{A} \left[\frac{B - 0}{QLG} - \frac{C - 0}{RMH} + \frac{F - 0}{SNH} - \frac{F - 0}{SNG} - \frac{J - 0}{TLG} + \frac{J - 0}{TMH} - \frac{J - 0}{TNH} + \frac{J - 0}{TNG} \right], \quad (3.2.45)$$

$$K_{102} = \sigma_{cP8} \sigma_{cN7} \sigma_{c6} \left[\frac{C - 0}{RMH} - \frac{F - 0}{SNH} + \frac{J - 0}{TNH} - \frac{J - 0}{THM} \right], \quad (3.2.46)$$

$$K_{107} = \frac{\sigma_{cP8} \sigma_{cN7}}{N} \left[\frac{F - 0}{S} - \frac{J - 0}{T} \right], \quad (3.2.47)$$

$$K_{108} = \frac{\sigma_{cP8}}{T} (J - 0) \quad (3.2.48)$$

$$K_{109} = 0, \quad (3.2.49)$$

$$0 = \exp(-\sigma_{a9} \theta), \quad (3.2.50)$$

$$Q = \sigma_{a9} - \sigma_{a5}, \quad (3.2.51)$$

$$R = \sigma_{a9} - \sigma_{a6}, \quad (3.2.52)$$

$$S = \sigma_{a9} - \sigma_{cN7}, \quad (3.2.53)$$

$$T = \sigma_{a9} - \sigma_{cP8}. \quad (3.2.54)$$

The remaining plutonium isotopes are Pu-240 and Pu-241. For the build up of Pu-240, I_0 , the differential equation to solve is

$$\frac{dI_0}{d\theta} = I_9 \sigma_{c9} - I_0 \sigma_{a0}. \quad (3.2.55)$$

The solution to Eq. (3.2.56) after substituting $I_{98} + I_{95}$ for I_9 is

$$I_0 = K_{51} I_5^0 + K_{53} I_8^0 + K_{54} (I_{98}^0 + I_{95}^0) + K_{55} I_0^0, \quad (3.2.56)$$

where

$$K_{51} = \frac{C5\sigma_{c9}}{D} \left[\frac{E - V}{W} + \frac{B - V}{X} \right], \quad (3.2.57)$$

$$K_{53} = C8\sigma_{c9} \left[\frac{1 - V}{\sigma_{a0}} + \frac{V - E}{W} \right], \quad (3.2.58)$$

$$K_{54} = \frac{\sigma_{c9}}{W} (E - V), \quad (3.2.59)$$

$$K_{55} = V, \quad (3.2.60)$$

$$V = \exp(-\sigma_{a0}\theta), \quad (3.2.61)$$

$$W = C9 + \sigma_{a0}, \quad (3.2.62)$$

$$X = \sigma_{a5} - \sigma_{a0}. \quad (3.2.63)$$

The concentration of Pu-241, I_1 , is formed from

$$\frac{dI_1}{d\theta} = I_0\sigma_{c0} - I_1\sigma_{a1}. \quad (3.2.64)$$

After substituting for I_0 , the solution is given as

$$I_1 = K_{61}I_5^0 + K_{63}I_8^0 + K_{64}(I_{98}^0 + I_{95}^0) + K_{65}I_0^0 + K_{66}I_1^0, \quad (3.2.65)$$

where

$$K_{61} = \frac{\sigma_{c0}\sigma_{c9}}{WDZ} \frac{C5}{(E - Y)} + \frac{B}{DX(AA)} - \frac{\sigma_{c0}\sigma_{c9}C5V}{(BB)Z} \left[\frac{1}{W} + \frac{1}{X} \right] \\ - \frac{\sigma_{c0}\sigma_{c9}C5Y}{D} \left[\frac{1}{X(AA)} - \frac{1}{W(BB)} - \frac{1}{X(BB)} \right], \quad (3.2.66)$$

$$K_{63} = \sigma_{c0}\sigma_{c9}C8 \left[\frac{1 - Y}{\sigma_{a1}\sigma_{a0}} + \frac{V - Y}{W(BB)} - \frac{V - Y}{\sigma_{a0}(BB)} - \frac{E - Y}{WZ} \right], \quad (3.2.67)$$

$$K_{64} = \sigma_{c0} \sigma_{c9} \left[\frac{E - Y}{WZ} - \frac{V - Y}{W(BB)} \right], \quad (3.2.68)$$

$$K_{65} = \frac{\sigma_{c0}}{(BB)} (V - Y), \quad (3.2.69)$$

$$K_{66} = Y, \quad (3.2.70)$$

$$Y = \exp(-\sigma_{a1} \theta), \quad (3.2.71)$$

$$Z = C9 + \sigma_{a1}, \quad (3.2.72)$$

$$AA = \sigma_{a1} - \sigma_{a5}, \quad (3.2.73)$$

$$BB = \sigma_{a1} - \sigma_{a0}. \quad (3.2.74)$$

Finally, the concentration for fission products, I_7 , is found by solving

$$\frac{dI_7}{d\theta} = 2(I_5 \sigma_{f5} + I_9 \sigma_{f9} + I_1 \sigma_{f1}). \quad (3.2.75)$$

Upon substituting for I_9 , the solution is

$$I_7 = K_{71} I_5^0 + K_{73} I_8^0 + K_{74} (I_{98}^0 + I_{95}^0) + K_{75} I_0^0 + K_{76} I_1^0, \quad (3.2.76)$$

where

$$\begin{aligned} K_{71} = & \frac{2C5(E - 1)}{C9D} \left[\sigma_{f9} + \frac{\sigma_{f1} \sigma_{c0} \sigma_{c9}}{WZ} \right] + \frac{2(B - 1)}{\sigma_{a5}} \left[\frac{\sigma_{f9} C5}{D} - \sigma_{f5} - \frac{\sigma_{f1} \sigma_{c0} \sigma_{c9} C5}{DX(AA)} \right] \\ & + \frac{2\sigma_{f1} \sigma_{c0} \sigma_{c9} C5}{\sigma_{a0} D(BB)} \left[\frac{V - 1}{W} + \frac{V - 1}{X} \right] + \frac{2\sigma_{f1} \sigma_{c0} \sigma_{c9} C5 (Y - 1)}{\sigma_{a1} D} \left[\frac{1}{WZ} + \frac{1}{X(AA)} \right. \\ & \left. - \frac{1}{W(BB)} - \frac{1}{X(BB)} \right], \end{aligned} \quad (3.2.77)$$

$$\begin{aligned}
K_{73} = & 2C8 \left[\sigma_{f9} + \frac{\sigma_{f1} \sigma_{c0} \sigma_{c9}}{\sigma_{a0} \sigma_{a1}} \right] \theta - 2C8(E - 1) \left[\frac{\sigma_{f9}}{C9} + \frac{\sigma_{f1} \sigma_{c0} \sigma_{c9}}{C9WZ} \right] \\
& + \frac{2\sigma_{f1} \sigma_{c0} \sigma_{c9} C8}{\sigma_{a1}} (Y - 1) \left[\frac{1}{\sigma_{a0} \sigma_{a1}} - \frac{1}{WZ} + \frac{1}{W(BB)} - \frac{1}{\sigma_{a0}(BB)} \right] \\
& + \frac{2\sigma_{f1} \sigma_{c0} \sigma_{c9} C8}{\sigma_{a0}(BB)} (V - 1) \left[\frac{1}{\sigma_{a0}} - \frac{1}{W} \right], \tag{3.2.78}
\end{aligned}$$

$$K_{74} = \frac{2\sigma_{f9}(E - 1)}{C9} \left[1 + \frac{\sigma_{c0} \sigma_{c9}}{WZ} \right] + \frac{2\sigma_{f1} \sigma_{c0} \sigma_{c9}}{W} \left[\frac{V - 1}{\sigma_{a0}(BB)} + \frac{Y - 1}{\sigma_{a1}Z} - \frac{Y - 1}{\sigma_{a1}(BB)} \right], \tag{3.2.79}$$

$$K_{75} = \frac{2\sigma_{f1} \sigma_{c0}}{(BB)} \left[\frac{Y - 1}{\sigma_{a1}} - \frac{V - 1}{\sigma_{a0}} \right], \tag{3.2.80}$$

$$K_{76} = \frac{2\sigma_{f1}}{\sigma_{a1}} (1 - Y). \tag{3.2.81}$$

The general form of the U-235 burnup for a fuel cycle of N irradiation periods is

$$I_{i5} = K_{11} I_{(i-1)5} + U_{i5} \quad i = 1, 2, \dots, N, \tag{3.2.82}$$

where U_{i5} is the amount of U-235 added to the reactor at the end of irradiation period i , $I_{(i-1)5}$ is the initial loading of irradiation period i , and I_{i5} is the initial amount of U-235 in the $(i+1)$ irradiation period. U_{N5} is zero because no U-235 will be added at the end of the last irradiation period. The concentration of U-235 as a function of fluence is shown in Fig. 3.2.1. U-235 and U-238 are the only nuclides considered which decrease during the irradiation period as shown in Fig. 3.2.1. The other nuclides build up during the irradiation period. The build up equation for U-236 is

$$I_{i6} = K_{21} I_{(i-1)5} + K_{22} I_{(i-1)6} - U_{i6}, \quad i = 1, 2, \dots, N, \tag{3.2.83}$$

where U_{i6} is the amount of U-236 removed at the end of fuel cycle i , and U_{N6} is zero. The concentration of U-236 as a function of fluence is shown in Fig. 3.2.2. For this model N was chosen as 5 years. Only the initial and final concentration of the nuclide during each irradiation period is of interest to this model.

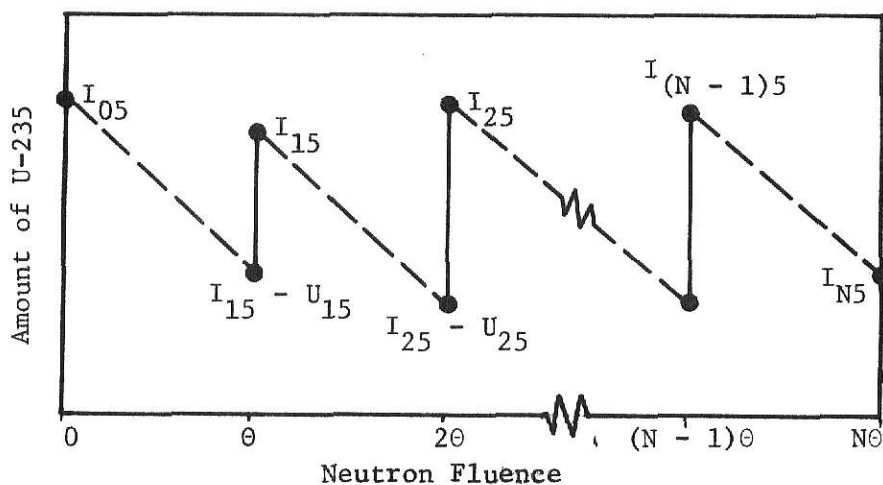


Figure 3.2.1 The burnup of U-235 during N irradiation periods.

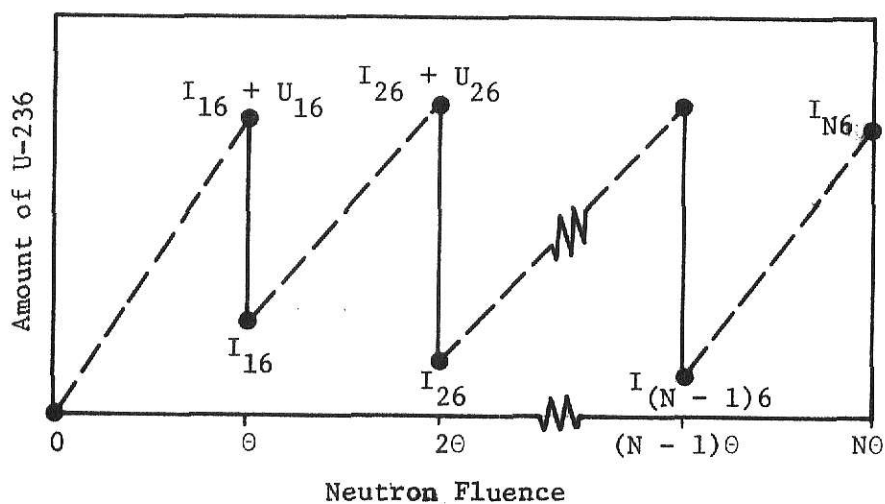


Figure 3.2.2 The buildup of U-236 during N irradiation periods.

3.3 Model Constraints

The nuclides are not entirely free to increase or decrease during the irradiation periods. Certain constraints must be satisfied and in order to satisfy them some of the nuclides must be added or removed at the end of each irradiation period. One such constraint, the multiplication factor constraint, must be equal to or greater than some minimum value at the beginning and end of each irradiation period. The multiplication factor, k_{eff} , is

$$k_{\text{eff}} = Pk_{\infty} = P(\eta\epsilon\text{pf}) , \quad (3.3.1)$$

$$k_{\text{eff}} = PK_1\eta , \quad (3.3.2)$$

where

$$K_1 = \epsilon\text{pf} , \quad (3.3.3)$$

P = non-leakage probability.

Equation (3.3.2) can be written as

$$\eta = \frac{k_{\text{eff}}}{PK_1} = \frac{\sum_{i=1}^{10} \nu_i \sigma_{fi} I_i}{\sum_{i=1}^{10} \sigma_{ai} I_i + \Sigma_s} , \quad (3.3.4)$$

where Σ_s is the macroscopic cross section representing all the materials present in the reactor except those mentioned in Section 3.2.

Equation (3.3.4) can be rearranged to the form

$$\begin{aligned} & Z_{11} I_{i5} + Z_{12} I_{i6} + Z_{13} I_{i8} + Z_{14} (I_{i98} + I_{i95}) + Z_{15} I_{i10} + Z_{16} I_{i11} + Z_{19} I_{iN7} + \\ & Z_{18} I_{iP8} + Z_{17} I_{i17} \geq Z_{110} , \end{aligned} \quad (3.3.5)$$

where

$$Z_{11} = \nu_5 \sigma_{f5} - \eta_0 \sigma_{a5} ,$$

$$Z_{12} = - \eta_0 \sigma_{a6} ,$$

$$Z_{13} = \eta_0 \sigma_{a8} ,$$

$$\begin{aligned}
Z_{14} &= \nu_9^{\sigma} f_9 - \eta_o^{\sigma} a_9 , \\
Z_{15} &= \quad \quad - \eta_o^{\sigma} a_0 , \\
Z_{16} &= \nu_1^{\sigma} f_1 - \eta_o^{\sigma} a_1 , \\
Z_{19} &= \quad \quad - \eta_o^{\sigma} a_{N7} , \\
Z_{18} &= \quad \quad - \eta_o^{\sigma} a_{P8} , \\
Z_{17} &= \quad \quad - \eta_o^{\sigma} a_7 , \\
Z_{110} &= \quad \quad \eta_o^{\Sigma} s ,
\end{aligned} \tag{3.3.6}$$

η_o = minimum value of η at the beginning of each irradiation period.

The corresponding equation for the end of each irradiation period is

$$\begin{aligned}
&Z_{21}(I_{i5} - U_{i5}) + Z_{22}(I_{i6} + U_{i6}) + Z_{23}(I_{i8} - U_{i8}) + Z_{24}(I_{i98} + U_{i98}) + \\
&Z_{24}(I_{i95} + U_{i95}) + Z_{25}(I_{i0} + U_{i0}) + Z_{26}(I_{i1} + U_{i1}) + Z_{29}(I_{iN7} + U_{iN7}) + \\
&Z_{28}(I_{iP8} + U_{iP8}) + Z_{27}(I_{i7} + U_{i7}) \geq Z_{210} ,
\end{aligned} \tag{3.3.7}$$

where

$$\begin{aligned}
Z_{21} &= \nu_5^{\sigma} f_5 - \eta_f^{\sigma} a_5 , \\
Z_{22} &= \quad \quad - \eta_f^{\sigma} a_6 , \\
Z_{23} &= \quad \quad - \eta_f^{\sigma} a_8 , \\
Z_{24} &= \nu_9^{\sigma} f_9 - \eta_f^{\sigma} a_9 , \\
Z_{25} &= \quad \quad - \eta_f^{\sigma} a_0 , \\
Z_{26} &= \quad \quad - \eta_f^{\sigma} a_1 , \\
Z_{29} &= \quad \quad - \eta_f^{\sigma} a_{N7} , \\
Z_{28} &= \quad \quad - \eta_f^{\sigma} a_{P8} , \\
Z_{27} &= \quad \quad - \eta_f^{\sigma} a_7 ,
\end{aligned} \tag{3.3.8}$$

$$Z_{210} = \eta_f \Sigma_s ,$$

η_f = minimum value of η at the end of each irradiation period.

The next constraint which must be satisfied is the fuel enrichment constraint. In most cases considered there will be a maximum enrichment constraint and in one specific case there will be a minimum enrichment constraint added. For the maximum enrichment constraint,

$$\frac{I_{i5}}{I_{i5} + I_{i8}} \leq En_{\max} , \quad (3.3.9)$$

which reduces to the form

$$Z_{41} I_{i5} + Z_{42} I_{i8} \leq 0 , \quad (3.3.10)$$

where

$$Z_{41} = 1 - En_{\max} , \quad (3.3.11)$$

$$Z_{42} = - En_{\max} . \quad (3.3.12)$$

For the minimum enrichment constraint the equation is

$$Z_{81} I_{i5} + Z_{82} I_{i8} \geq 0 , \quad (3.3.13)$$

where

$$Z_{81} = 1 - En_{\min} , \quad (3.3.14)$$

$$Z_{82} = - En_{\min} . \quad (3.3.15)$$

Another constraint to the model is the volume constraint. The volume of the nuclides in the core cannot exceed the volume of the core, which was assumed to be approximately the volume of the fuel at the initial loading. The volume constraint is then written as

$$Z_{61} I_{i5} + Z_{62} I_{i6} + Z_{63} I_{i8} + Z_{64} (I_{i98} + I_{i95}) + Z_{65} I_{i10} + Z_{66} I_{i11} + Z_{69} I_{iN7} + \\ Z_{68} I_{iP8} + Z_{67} I_{i17} \leq V , \quad (3.3.16)$$

where Z_{6j} is the volume coefficient, in units of liter/kgm-atom, of the

corresponding nuclides, and V is the assumed volume of the core in liters.

The final restrictions on the model will be those which control the amount of certain nuclides which will be added to or removed from the reactor during refueling.

Since it is desired to minimize the amount of U-235 added to the reactor the objective function will be

$$\min Z = I_{55} + U_{15} + U_{25} + U_{35} + U_{45} . \quad (3.3.17)$$

This has a dual purpose: Not only does it minimize the amount of U-235 added during the refueling periods but also it obtains the best burnup of the fuel in the reactor by minimizing the amount of U-235 left after all the irradiation periods.

3.4 Model Formulation

The burnup equations and constraint equations must be rewritten for each irradiation period. A typical set of equations for any irradiation period will be similar to those for the first irradiation period:

$$\begin{aligned}
 K_{11}I_{05} - I_{15} + U_{15} &= 0, \\
 K_{21}I_{05} + K_{22}I_{06} - I_{16} - U_{16} &= 0, \\
 K_{33}I_{08} - I_{18} + U_{18} &= 0, \\
 K_{41}I_{05} + K_{43}I_{08} + K_{44}I_{098} - I_{198} - U_{198} &= 0, \\
 K_{101}I_{05} + K_{102}I_{06} + K_{109}I_{095} + K_{107}I_{0N7} + K_{108}I_{0P8} - I_{195} - U_{195} &= 0, \\
 K_{51}I_{05} + K_{53}I_{08} + K_{54}I_{098} + K_{54}I_{095} + K_{55}I_{00} - I_{10} - U_{10} &= 0, \\
 K_{61}I_{05} + K_{63}I_{08} - K_{64}I_{098} + K_{64}I_{095} + K_{65}I_{00} + K_{66}I_{01} - I_{11} - U_{11} &= 0, \\
 K_{91}I_{05} + K_{92}I_{06} + K_{97}I_{0N7} - I_{1N7} - U_{1N7} &= 0, \quad (3.4.1) \\
 K_{81}I_{05} + K_{82}I_{06} + K_{87}I_{0N7} + K_{88}I_{0P8} - I_{1P8} - U_{1P8} &= 0, \\
 K_{71}I_{05} + K_{73}I_{08} + K_{74}I_{098} + K_{74}I_{095} + K_{75}I_{00} + K_{76}I_{01} - I_{17} - U_{17} &= 0, \\
 Z_{11}I_{15} + Z_{12}I_{16} + Z_{13}I_{18} + Z_{14}I_{198} + Z_{14}I_{195} + Z_{15}I_{10} + Z_{16}I_{11} + Z_{19}I_{1N7} + \\
 Z_{18}I_{1P8} + Z_{17}I_{17} &\geq Z_{110}, \\
 Z_{21}(I_{15} - U_{15}) + Z_{22}(I_{16} + U_{16}) + Z_{23}(I_{18} - U_{18}) + Z_{24}(I_{198} + U_{198}) + \\
 Z_{24}(I_{195} + U_{195}) + Z_{25}(I_{10} + U_{10}) + Z_{26}(I_{11} + U_{11}) + Z_{29}(I_{1N7} + U_{1N7}) + \\
 Z_{28}(I_{1P8} + U_{1P8}) + Z_{27}(I_{17} + U_{17}) &\geq Z_{210}, \\
 Z_{41}I_{15} + Z_{42}I_{18} &\leq 0, \\
 Z_{61}I_{15} + Z_{62}I_{16} + Z_{63}I_{18} + Z_{64}I_{198} + Z_{64}I_{195} + Z_{65}I_{10} + Z_{66}I_{11} + Z_{69}I_{1N7} + \\
 Z_{68}I_{1P8} + Z_{67}I_{17} &\leq V,
 \end{aligned}$$

Other equations can be added to constrain the model further. For example, to make sure that all the U-236 is removed after the first irradiation period the equation $I_{16} = 0$ is added. It can be observed then from Eq. (3.4.1) that

$$U_{16} = K_{21}I_{05} + K_{22}I_{06}, \quad (3.4.2)$$

where U_{16} , the amount of U-236 removed at the end of the first irradiation period, is equal to the amount of U-236 produced during the first irradiation period. To make sure that no U-236 is removed at the end of the irradiation period add the equation

$$U_{16} = 0. \quad (3.4.3)$$

Similar equations can be added to restrict the addition or removal of other nuclides. All of the equations can be put into the mini-matrix form of Fig. 3.4.1. The complete matrix is shown on page 13 of the MPS Computer Program listing in Appendix B.

		Objective Function				N	Zero
1 ↓ 10	Initial Loading					=	Zero
11 ↓ 20	1st Year Irradiation Period	Refuel Var. 1st Year				=	Zero
21 ↓ 30	2nd Year Irradiation Period	Refuel Var. 2nd Year				=	Zero
31 ↓ 40	3rd Year Irradiation Period	Refuel Var. 3rd Year				=	Zero
41 ↓ 50	4th Year Irradiation Period	Refuel Var. 4th Year				=	Zero
51 ↓ 60	5th Year Irradiation Period					=	Zero
61 ↓ 64	$k_{eff} \geq 1.08$					$\geq$	.39941
65 ↓ 69		$k_{eff} \geq 1.0001$				$\geq$	.36986
70 ↓ 73						$\leq$	Zero
74 ↓ 77	Maximum Enrichment Constraint					$\leq$	Zero
78 ↓ 82	Minimum Enrichment Constraint (Optional)					$\geq$	Zero
83 ↓ 114	Volume Constraints					$\leq$	3900
	Nuclide Removal and Loading Constraints					=	Zero

Figure 3.4.1 The mini-matrix of the linear programming nuclear reactor refueling model.

4.0 RESULTS OF MODEL ANALYSIS

4.1 Evaluation of Constants

The reactor was assumed to operate at 1000 Mwe with an average flux of $5(10^{13})$ neutrons/cm²sec. The irradiation period was chosen as 360 days, therefore, from Eq. (3.2.1), θ is 1.5552 neutrons/kb. With that value of θ and the data given in Table 4.1.1 the constants in the burnup equations were easily calculated and are given in Table 4.1.2.

The constants represented in Eqs. (3.3.5 - 3.3.8) were calculated after η_o , η_f , Σ_s were defined. The minimum values of k_{eff} at the beginning and end of each irradiation period were chosen arbitrarily, but in agreement with Tabak (3), as 1.08 and 1.0001, respectively. P was assumed to be 0.80 and K_1 was assumed to be 0.85 (5). Therefore, from Eq. (3.3.4) η_o and η_f were calculated to be 1.58824 and 1.4707, respectively. Σ_s was calculated from data given for a similar reactor (6) and was found to be $.25148 \text{ cm}^{-1}$. The calculated constants are listed in Table 4.1.3.

The reactor was initially loaded with 10.6383 kg-atoms of U-235 and 297.89916 kg-atoms of U-238, which corresponds to an initial enrichment of 3.448%. For the enrichment constraints in Section 3.3, En_{max} was chosen as 3.5% and En_{min} was chosen as 3.0%. Therefore from Eqs. (3.3.9 - 3.3.15), $Z_{41} = .965$, $Z_{42} = -.035$, $Z_{81} = .97$, and $Z_{82} = -.03$.

The constants for the volume constraints were calculated and are listed in Table 4.1.4. For the initial loading the volume of the fuel was calculated to be 3873.35 liters. Hence, the maximum volume of the fuel was fixed at $V = 3900$ liters.

Table 4.1.1 Cross Section Data

Nuclide	$\sigma_a(\text{kb})^\dagger$	$\sigma_c(\text{kb})^\dagger$	$\sigma_f(\text{kb})^\dagger$	ν^*	η^*
U-235	.6780	.0980	.580	2.47	1.97
U-236	.0055	.0055			
U-238	.00273	.00273			
Pu-238	.5400	.5400			
Pu-239	1.0130	.2710	.742	2.905	1.81
Pu-240	.2800	.2800			
Pu-241	1.3640	.3640	1.000	3.06	
Np-237	.1690	.1690			
fission product	.0319				

† Data taken from the Chart of the Nuclides, USAEC, May 1970.

* Data taken from revised Chapter 3 of reference 6.

Table 4.1.2 Burnup Equation Constants

Constant	Value	Constant	Value	Constant	Value
K ₁₁	.34839	K ₁₀₈	.25673	K ₉₂	.00749
K ₂₁	.09371	K ₅₁	.05026	K ₉₇	.76887
K ₂₂	.99148	K ₅₃	.00059	K ₈₁	.00003
K ₃₃	.99576	K ₅₄	.22178	K ₈₂	.00079
K ₄₁	.19893	K ₅₅	.64697	K ₈₇	.15355
K ₄₃	.00284	K ₆₁	.00537	K ₈₈	.43179
K ₄₄	.42152	K ₆₃	.00006	K ₇₁	1.44436
K ₁₀₁	.00001	K ₆₄	.03117	K ₇₃	.00380
K ₁₀₂	.00017	K ₆₅	.13615	K ₇₄	1.77141
K ₁₀₄	.20692	K ₆₆	.11988	K ₇₅	.31801
K ₁₀₇	.04684	K ₉₁	.00043	K ₇₆	1.29050

Table 4.1.3 Multiplication Factor Constraint Constants

Constant	Value	Constant	Value
z_{11}	.35578	z_{21}	.43544
z_{12}	- .00874	z_{22}	- .00809
z_{13}	- .00434	z_{23}	- .00402
z_{14}	.54663	z_{24}	.66566
z_{15}	- .44471	z_{25}	- .41181
z_{16}	.89365	z_{26}	1.05392
z_{19}	- .26841	z_{29}	- .24855
z_{18}	- .85765	z_{28}	- .79420
z_{17}	- .05066	z_{27}	- .04692
z_{110}	.39941	z_{210}	.36986

Table 4.1.4 Volume Constraint Constants

Constant	Value 1/kgm-atom	Constant	Value 1/kgm-atom
Z_{61}	12.401	Z_{66}	12.147
Z_{62}	12.454	Z_{69}	12.474
Z_{63}	12.559	Z_{68}	11.996
Z_{64}	12.046	Z_{67}	1.7187
Z_{65}	12.097	V	3900.00

4.2 Refueling Schemes

Twelve refueling schemes were considered. Two such schemes included no plutonium recycle, one with no minimum enrichment constraint and the other with a minimum enrichment of 3%. The other ten schemes considered different degrees of plutonium recycle varying from 10% to 100% by increments of 10% with no minimum enrichment constraint. The nuclide flow diagram of Fig. 4.2.1 shows the general flow of the nuclides from one irradiation period to the next for a fuel cycle of five irradiation periods. The amount of plutonium removed from the system depends on the percent of plutonium recycle. When plutonium recycle is considered, each isotope of plutonium is recycled at the same percent. This is due to their identical chemical and similar physical properties which constrains their separation. All of the U-236 is recycled because it cannot be easily separated from U-235 due to their identical chemical properties.

As shown in Fig. 4.2.1, all of the Np-237 and fission products are removed after each irradiation period, which emphasizes the assumption of complete reprocessing of the reactor core after each irradiation period. This assumption also considers that the reprocessing and refueling is completed in an amount of time which is insignificant when compared to the year operating time. Another assumption made is that U-235 can be added to the reactor in a pure form, i.e., 100% enriched uranium.

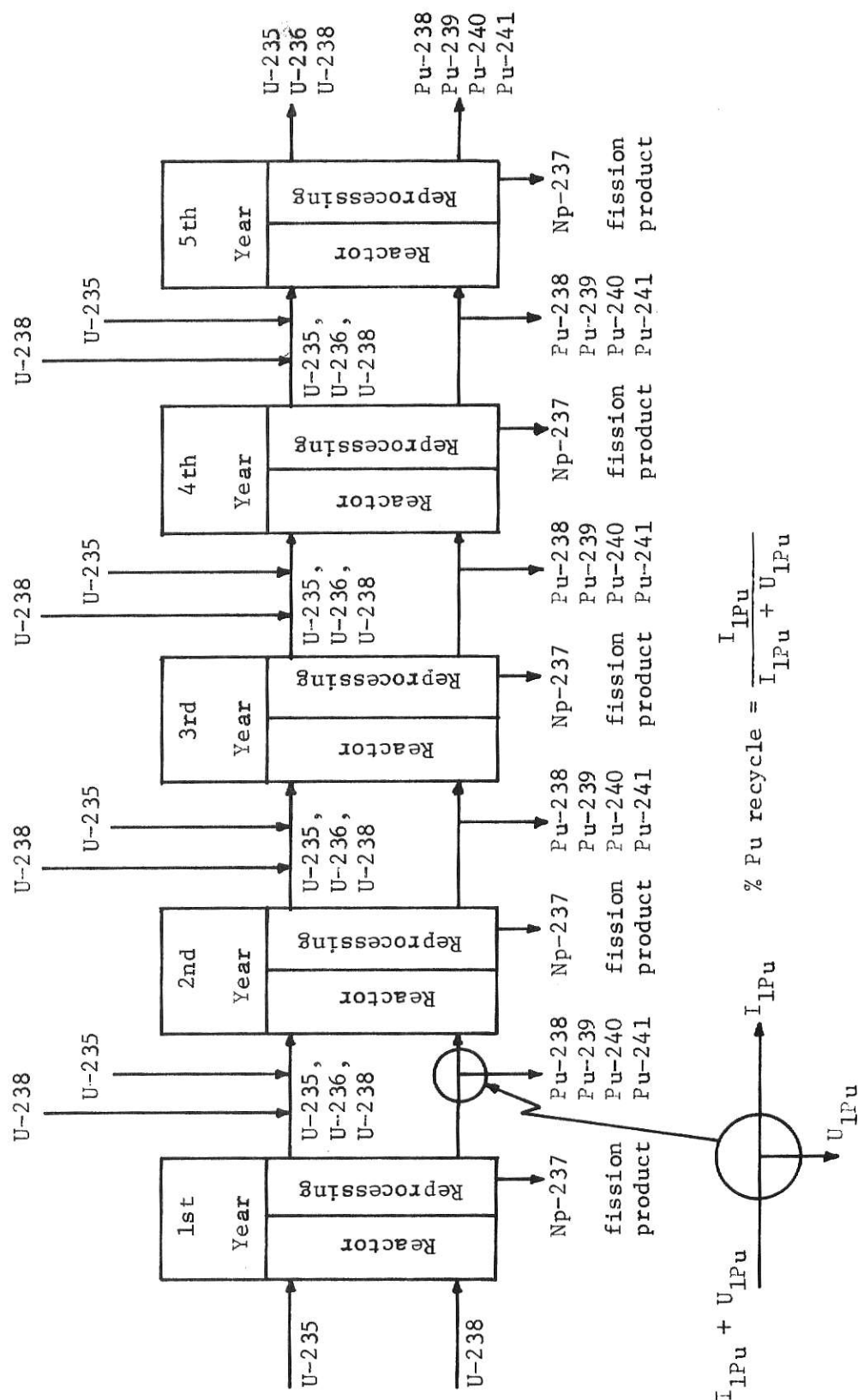


Figure 4.2.1 Nuclide Flow Diagram for a Five Irradiation Period Reactor Fuel Cycle

4.3 Graphical Results

The IBM-MPS/360 which uses the revised simplex method of solving LP problems was used to obtain the solutions for the cases considered.

Figures 4.3.1 through 4.3.24 are plots of the results obtained for the 12 refueling cases considered.

Figures 4.3.1, 4.3.2, and 4.3.3 are plots of the kgm-atoms of U-235 versus the time of operation of the reactor for the three different cases of 0%, 50%, and 100% Pu recycle. Figures 4.3.4 through 4.3.11 are plots of the amount of the eight other nuclides versus the years of reactor operation for the case of 50% Pu recycle. Plots of the total amount of plutonium produced in and removed from the reactor versus the time of reactor operation for the cases of 0%, 50%, and 100% Pu recycle are shown in Figs. 4.3.12 through 4.3.14. These values were found by summing the contributions from each plutonium isotope at each refueling time. Figures 4.3.15, 4.3.16 and 4.3.17 are plots of the amounts of fissionable plutonium versus the time of reactor operation for the case of 0%, 50%, and 100% Pu recycle.

Figure 4.3.18 is a plot of the objective function, i.e., the total U-235 requirements, versus the percent of Pu recycled. The amount of U-235 added to the reactor during the fuel cycle, which is found by subtracting the amount of U-235 left at the end of the fuel cycle from the value of the objective function, versus the percent of Pu recycle is shown in Fig. 4.3.19. Figure 4.3.20 shows the total amount of plutonium recycled and the amount of fissionable plutonium recycled versus the percent of Pu recycle. Figure 4.3.21 shows only the fissionable plutonium recycled and the individual contributions of Pu-239 and Pu-241 to this curve. Figures 4.3.22 and 4.3.23 each show the ratio of the change in the U-235 inventory to the change in the total and fissionable plutonium inventory. The change in inventories plotted in

Fig. 4.3.22 was found by using the 0% Pu recycle case as the reference. In Fig. 4.3.23 the change was found by using the previous percent Pu recycle case as the reference to the case being considered. The final figure, Fig. 4.3.24, shows the amount of fission product pairs removed during the fuel cycle for the different Pu recycle cases considered.

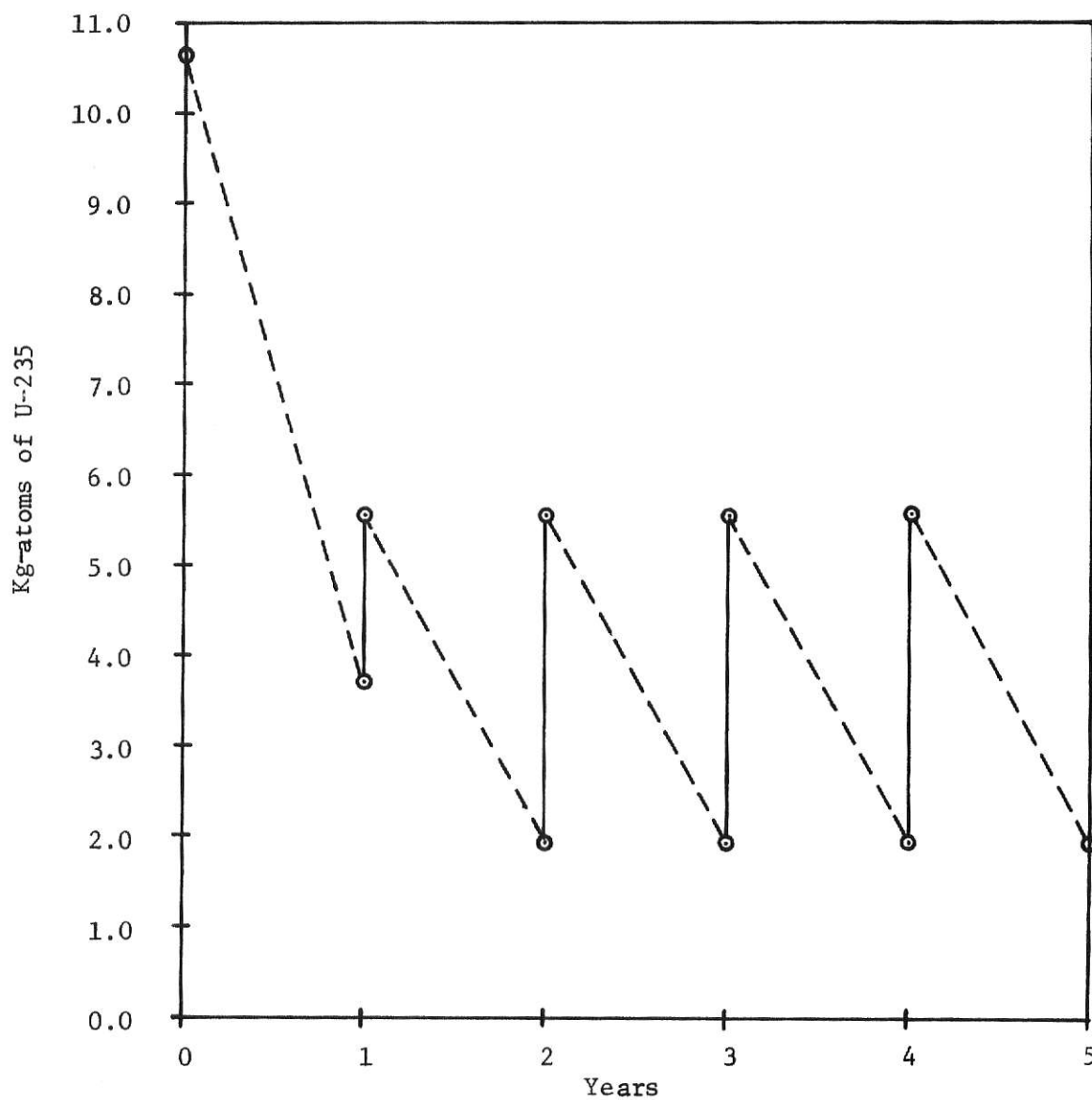


Figure 4.3.1 The amount of U-235 in the reactor during the fuel cycle vs the number of irradiation periods for the 0% Pu recycle case.

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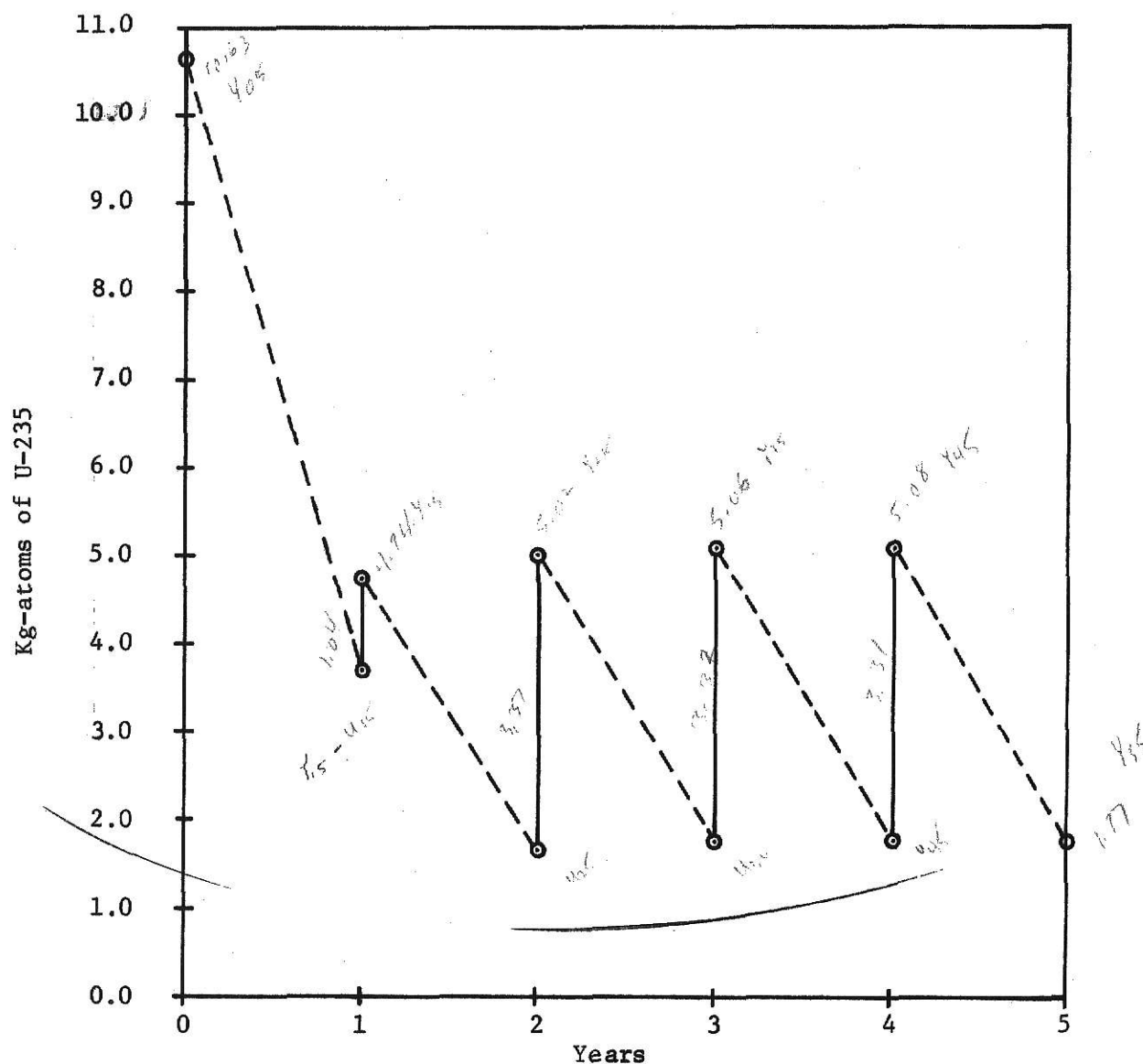


Figure 4.3.2 The amount of U-235 in the reactor during the fuel cycle vs the number of irradiation periods for the 50% Pu recycle case.

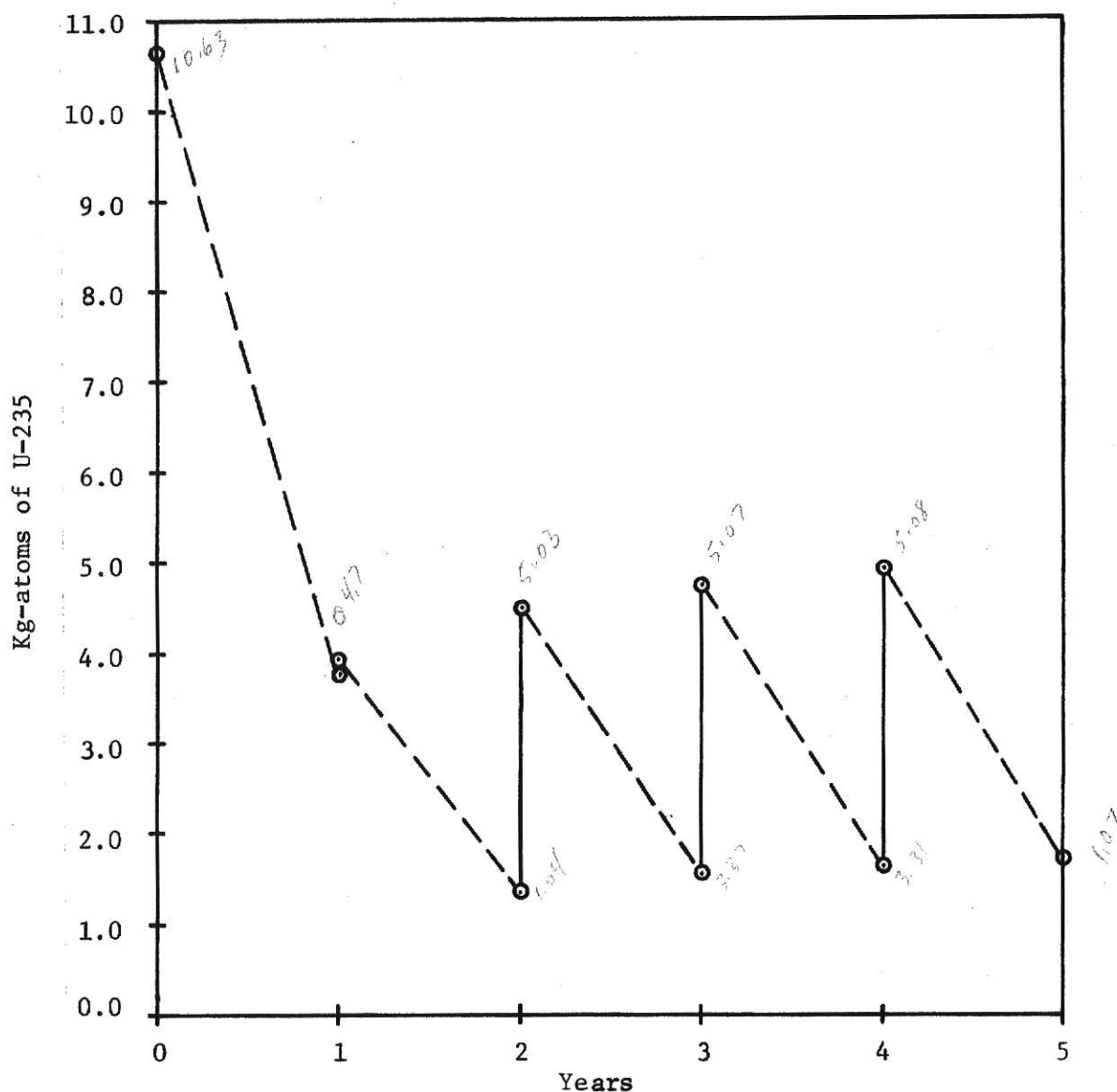


Figure 4.3.3 The amount of U-235 in the reactor during the fuel cycle vs the number of irradiation periods for the 100% Pu recycle case.

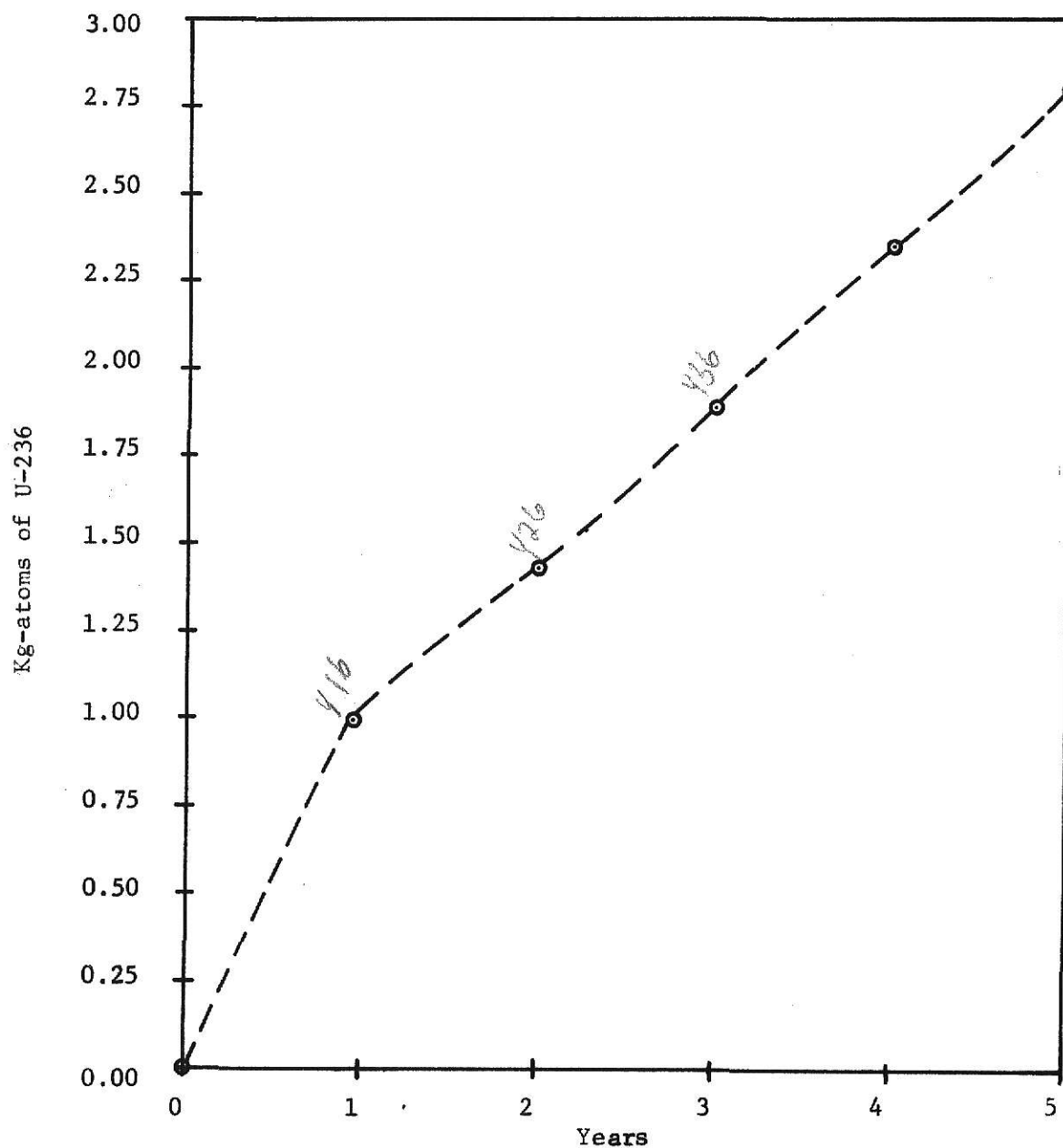


Figure 4.3.4 The amount of U-236 in the reactor during the fuel cycle vs the number of irradiation periods for the 50% Pu recycle case.

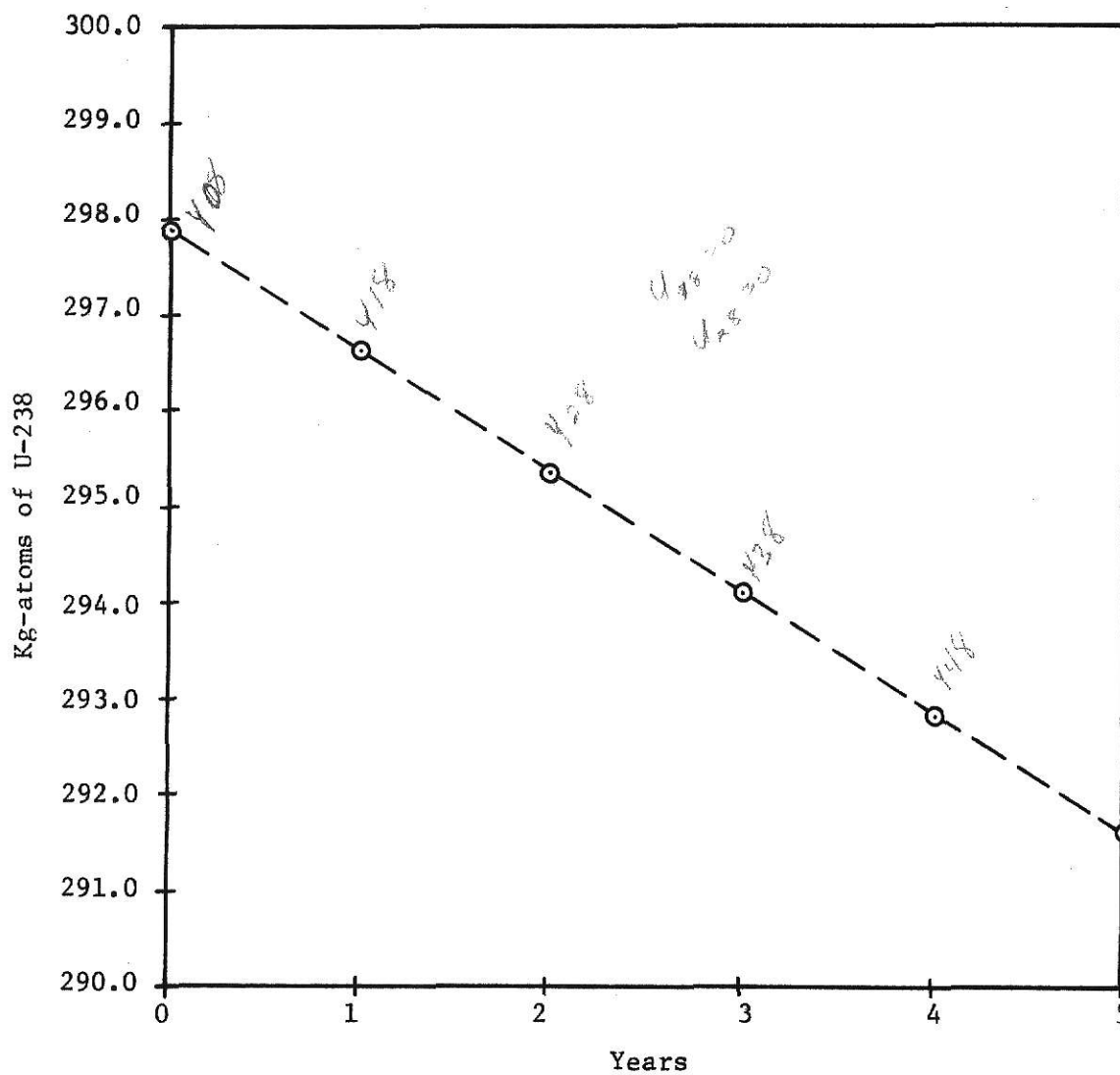


Figure 4.3.5 The amount of U-238 in the reactor during the fuel cycle vs the number of irradiation periods for the 50% Pu recycle case.

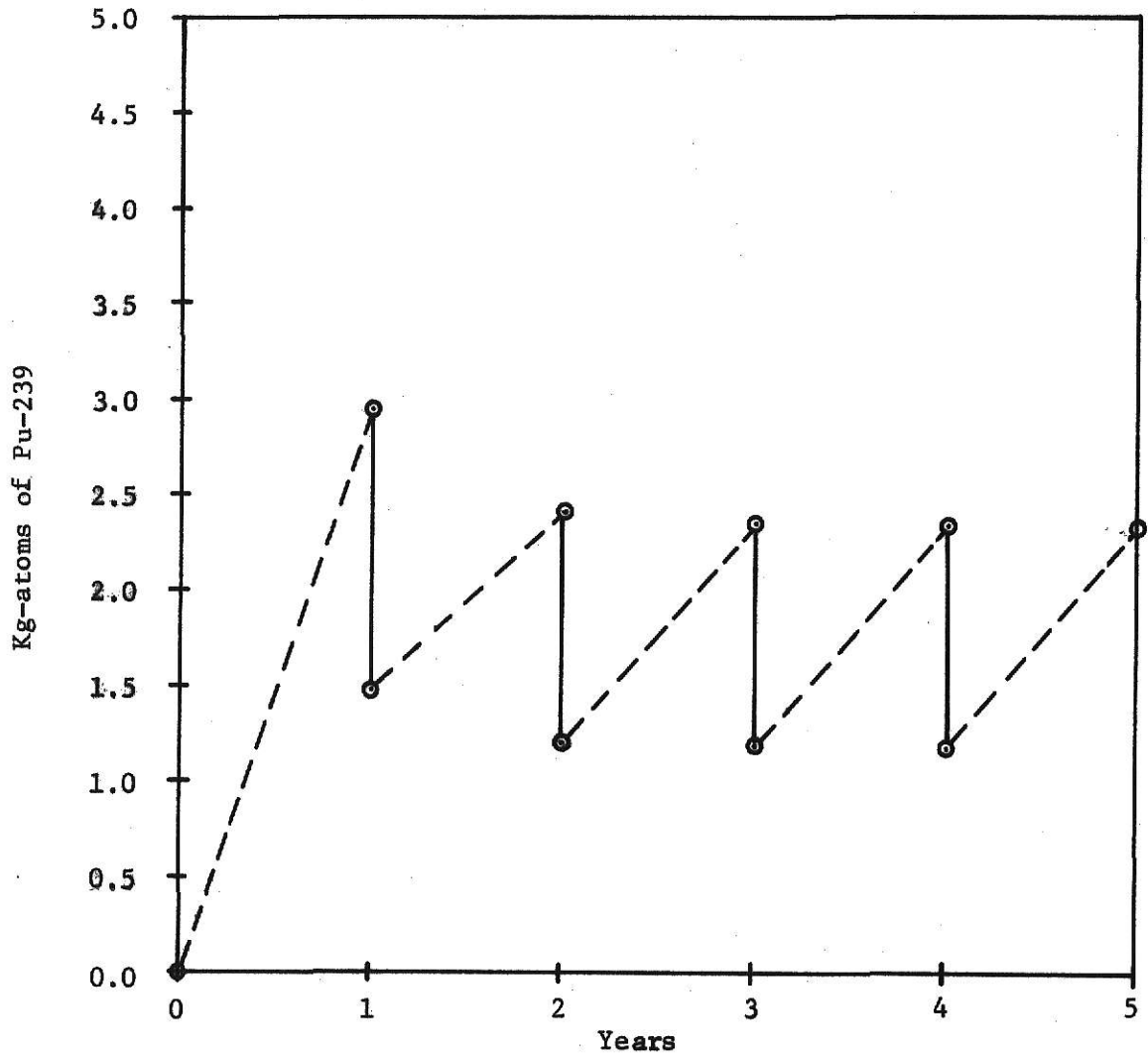


Figure 4.3.6 The amount of Pu-239 in the reactor during the fuel cycle vs the number of irradiation periods for the 50% Pu recycle case.

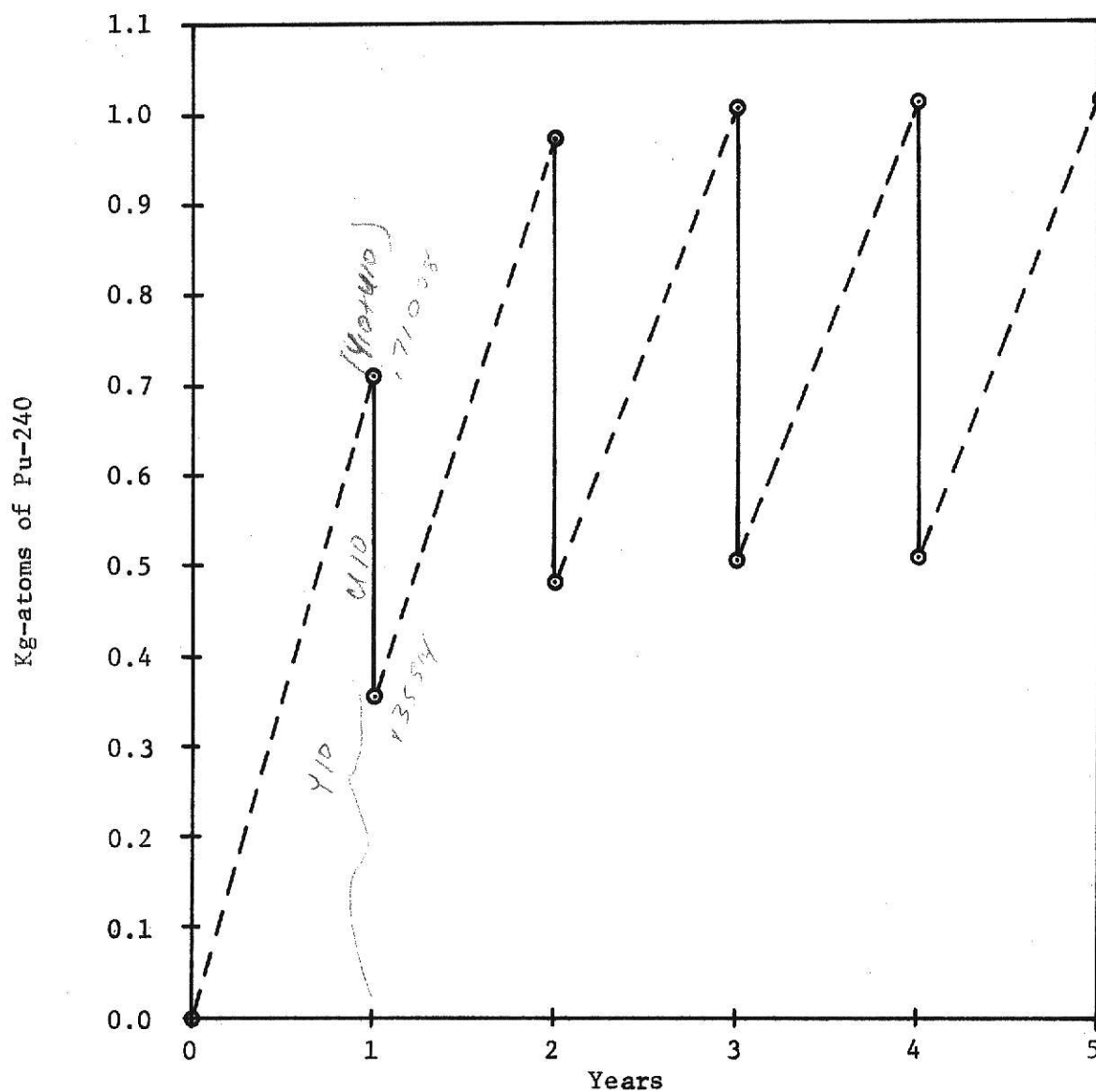


Figure 4.3.7 The amount of Pu-240 in the reactor during the fuel cycle vs the number of irradiation periods for the 50% Pu recycle case.

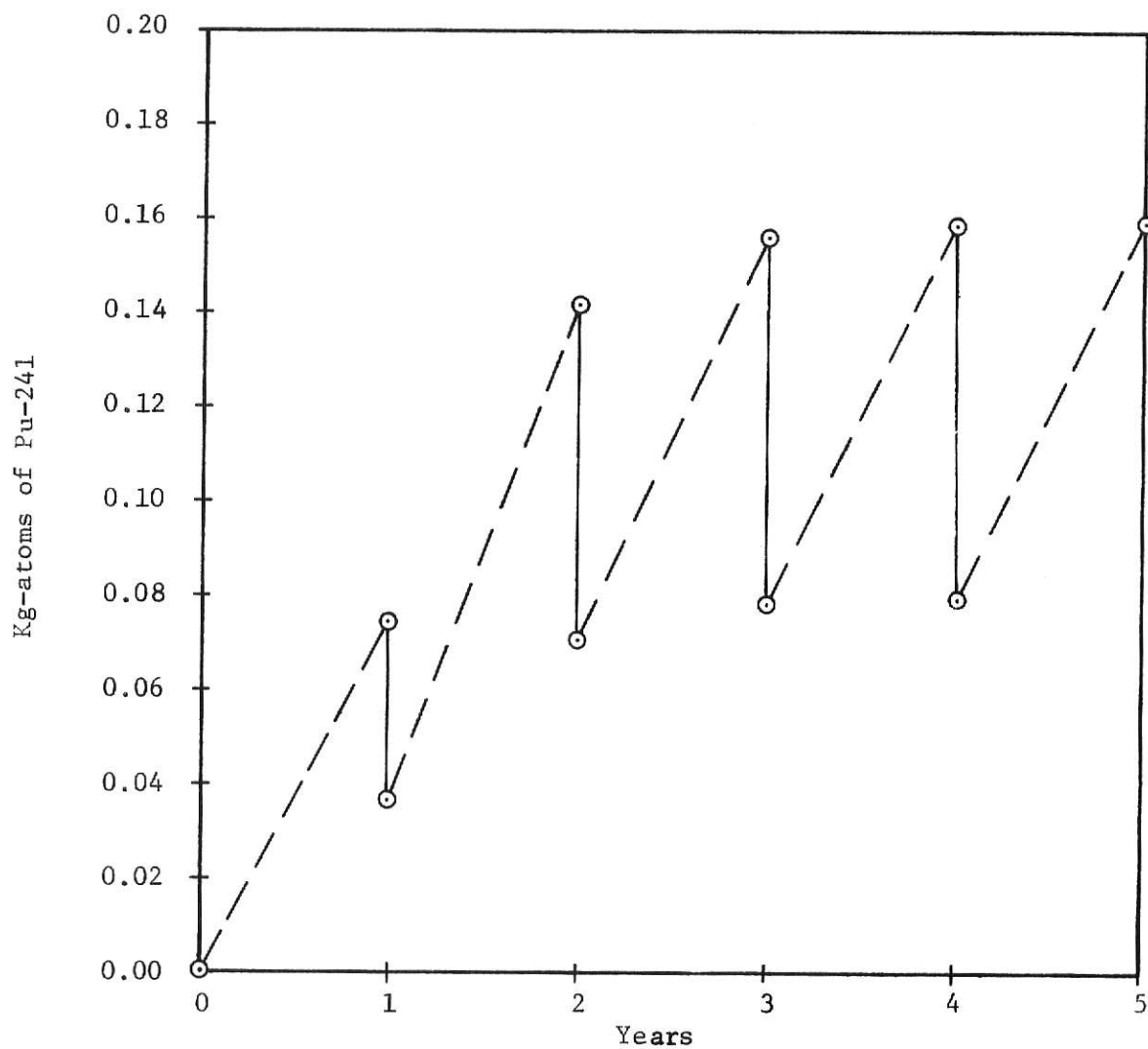


Figure 4.3.8 The amount of Pu-241 in the reactor during the fuel cycle vs the number of irradiation periods for the 50% Pu recycle case.

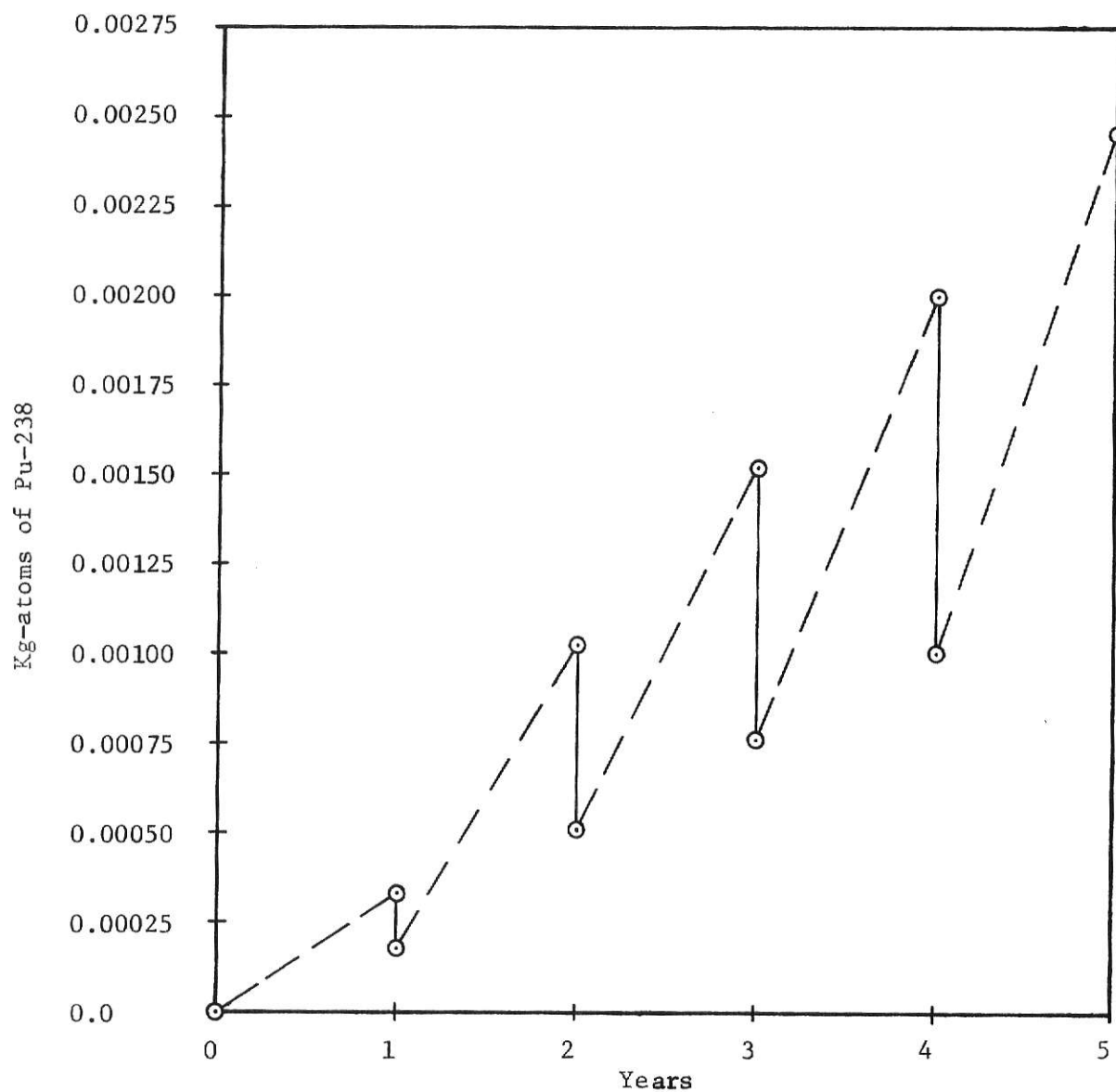


Figure 4.3.9 The amount of Pu-238 in the reactor during the fuel cycle vs the number of irradiation periods for the 50% Pu recycle case.

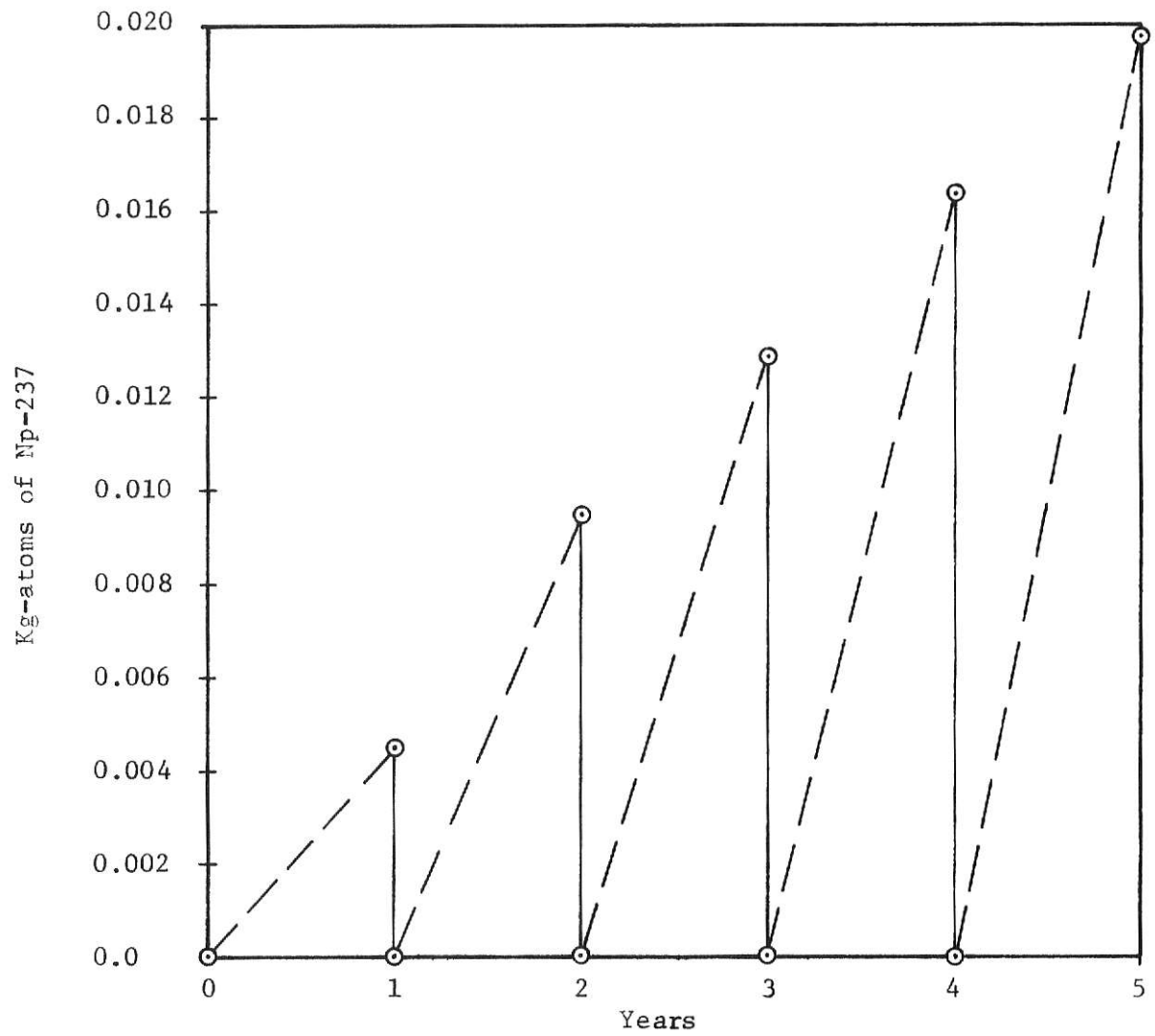


Figure 4.3.10 The amount of Np-237 in the reactor during the fuel cycle vs the number of irradiation periods for the 50% Pu recycle case.

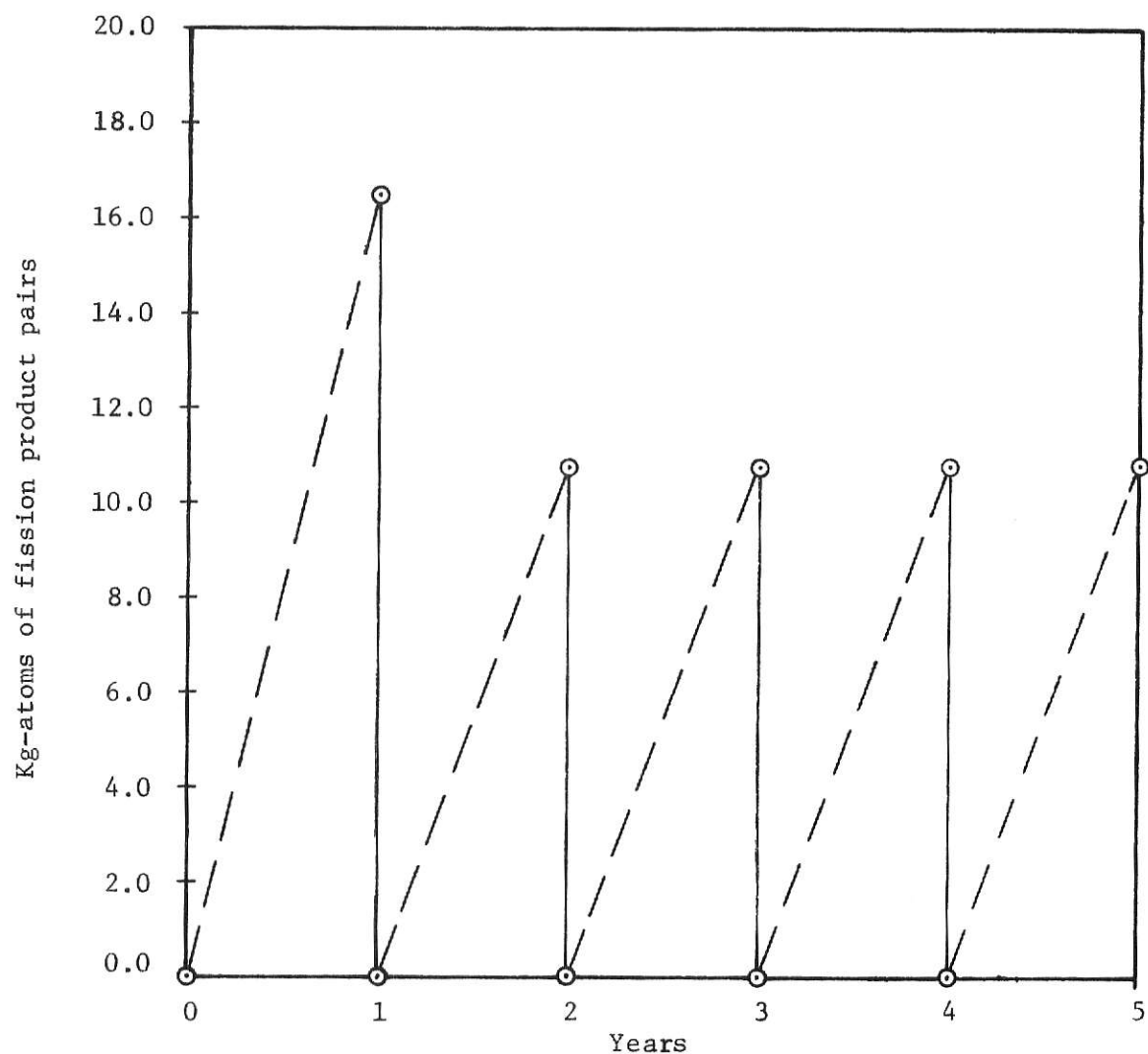


Figure 4.3.11 The amount of fission product pairs in the reactor during the fuel cycle vs the number of irradiation periods for the 50% Pu recycle case.

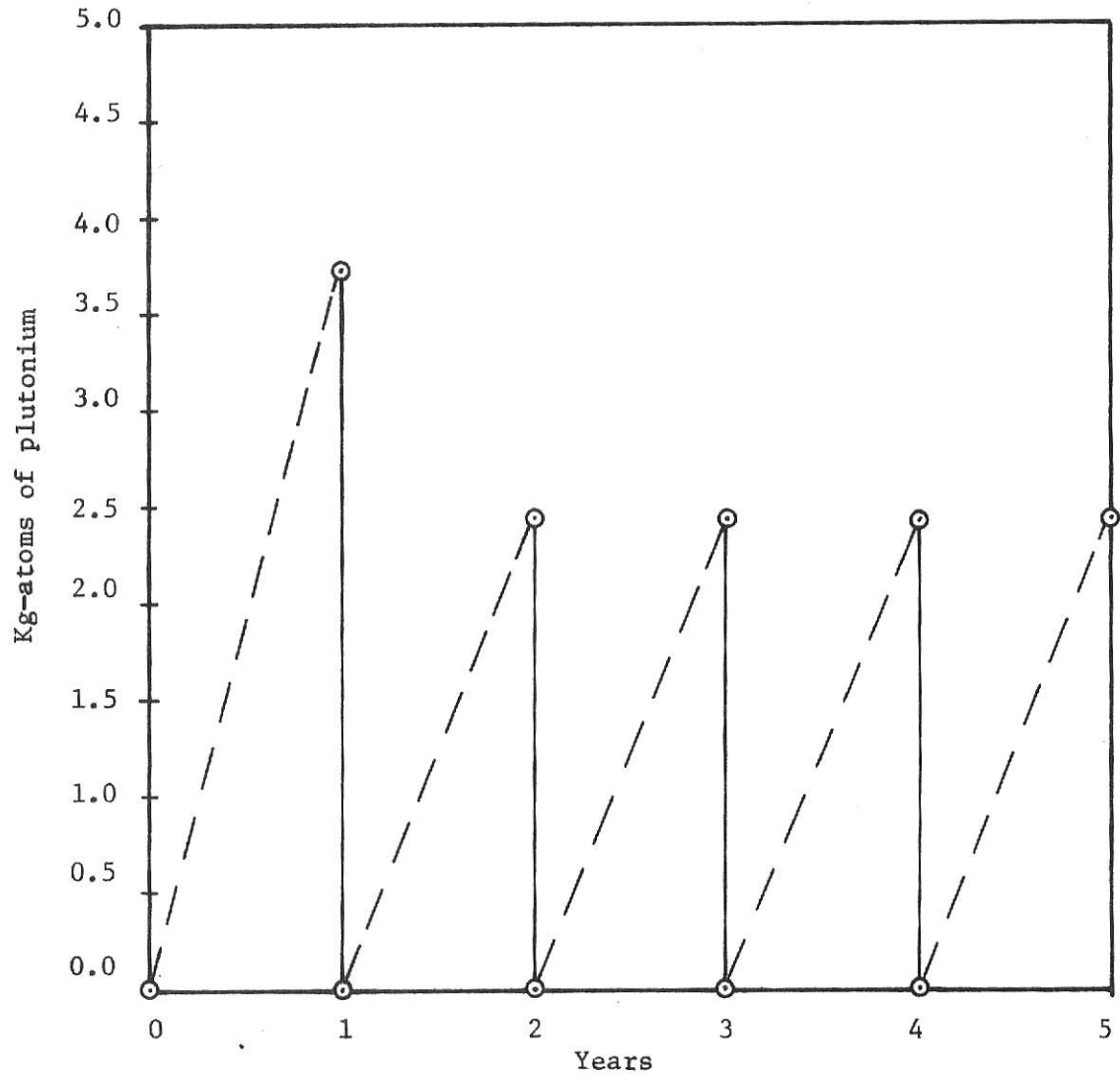


Figure 4.3.12 The total amount of plutonium in the reactor during the fuel cycle vs the number of irradiation periods for the 0% Pu recycle case.

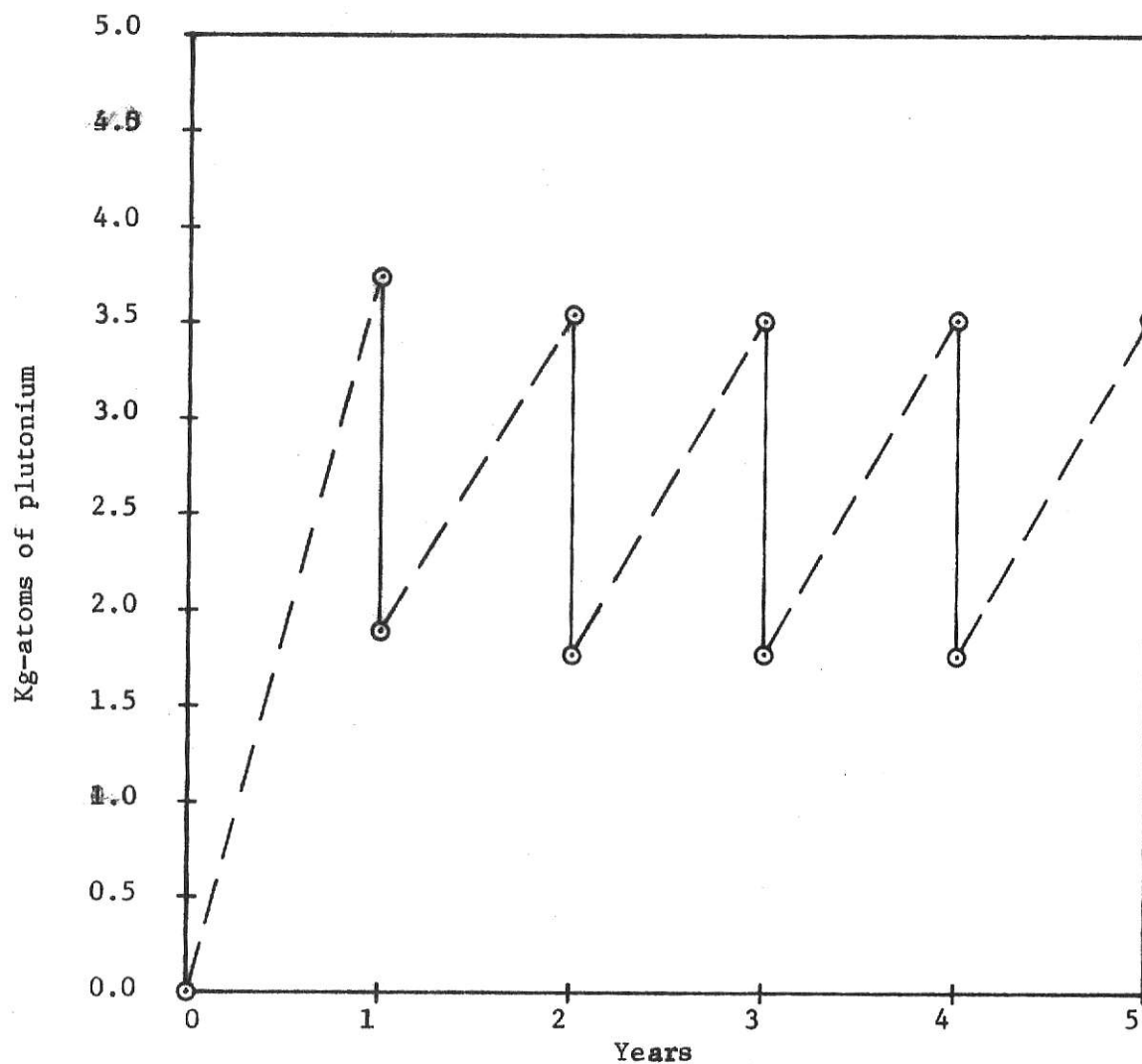


Figure 4.3.13 The total amount of plutonium in the reactor during the fuel cycle vs the number of irradiation periods for the 50% Pu recycle case.

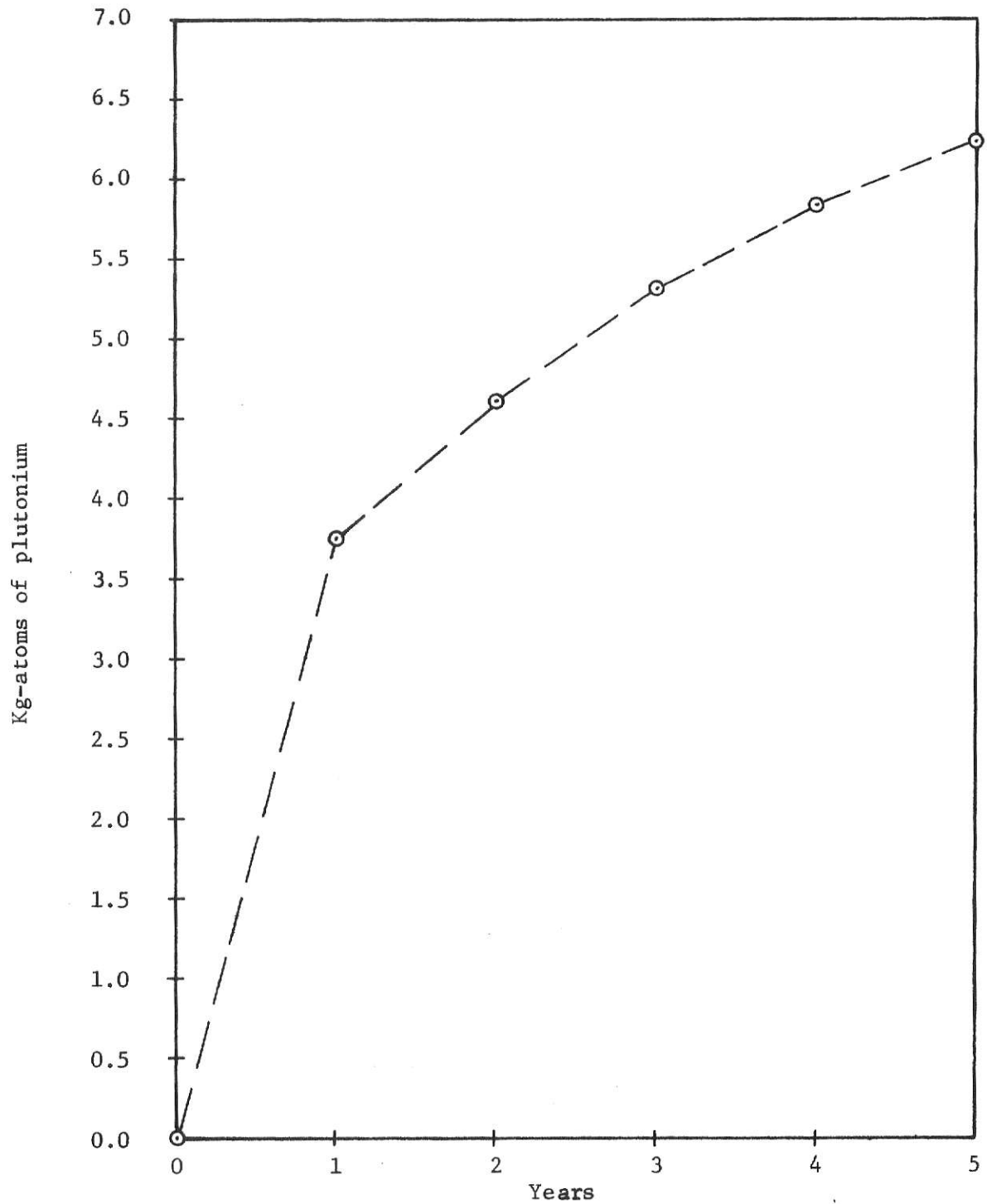


Figure 4.3.14 The total amount of plutonium in the reactor during the fuel cycle vs the number of irradiation periods for the 100% Pu recycle case.

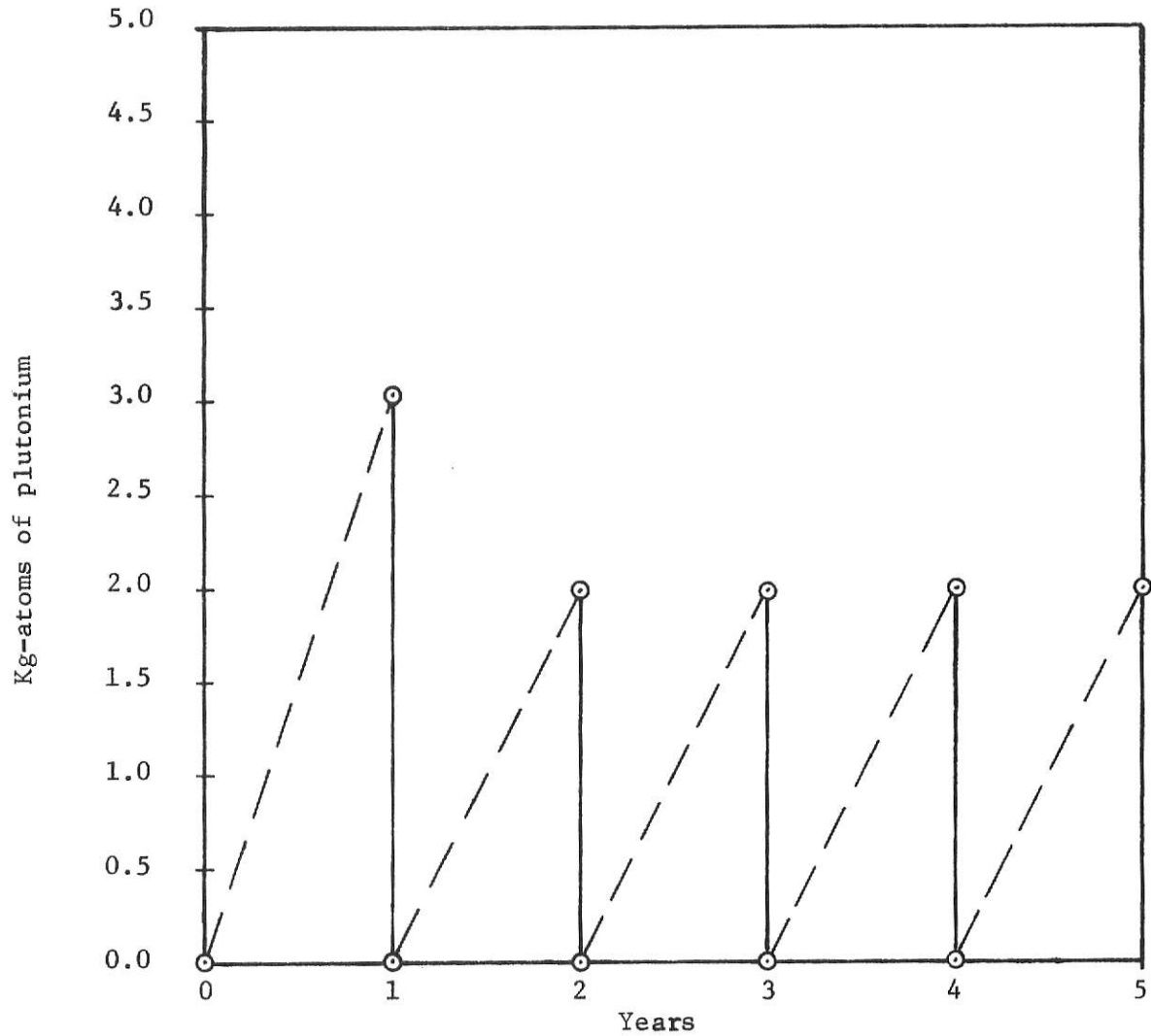


Figure 4.3.15 The amount of fissionable plutonium in the reactor during the fuel cycle vs the number of irradiation periods for the 0% Pu recycle case.

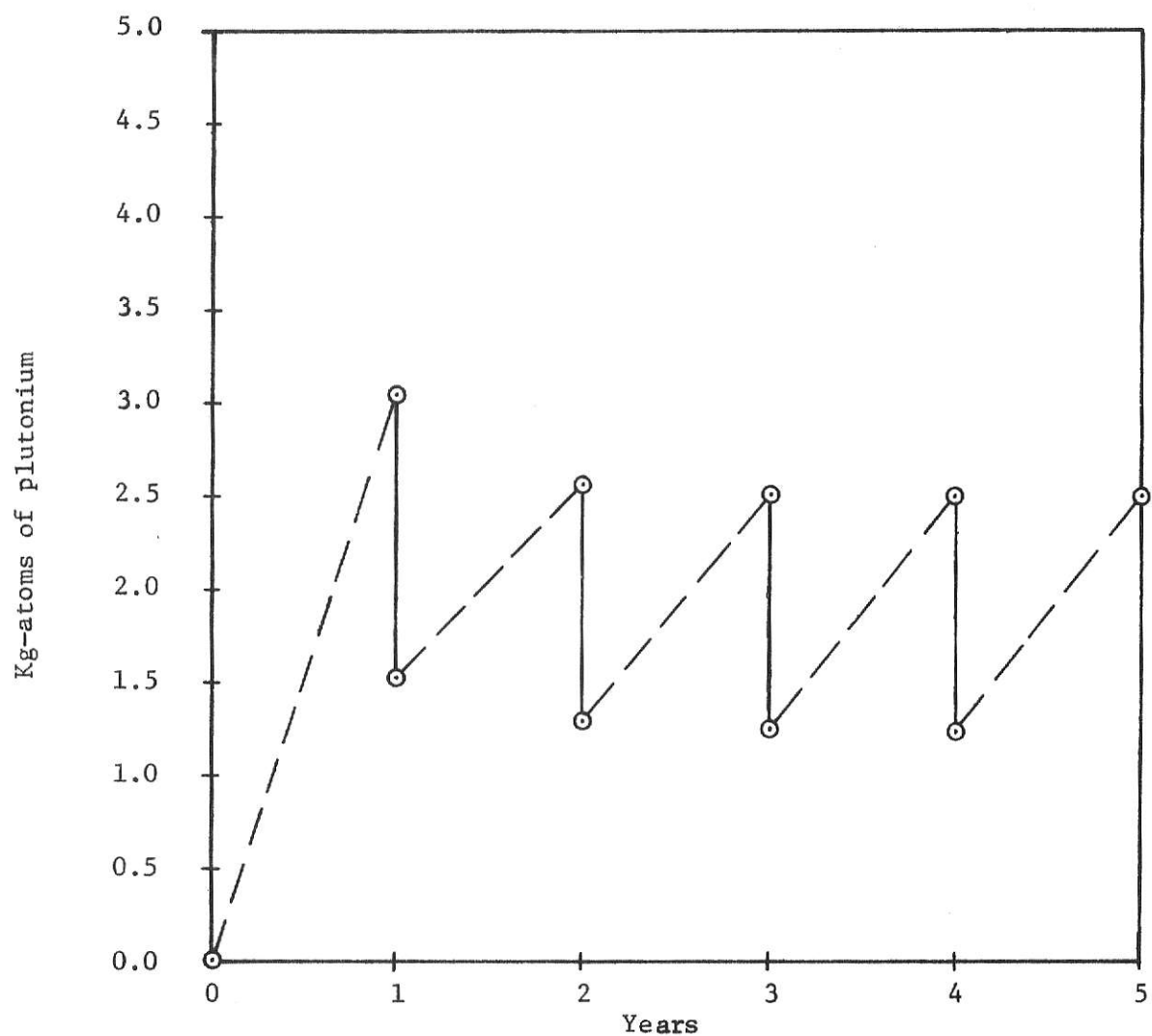


Figure 4.3.16 The amount of fissionable plutonium in the reactor during fuel cycle vs the number of irradiation periods for the 50% Pu recycle case.

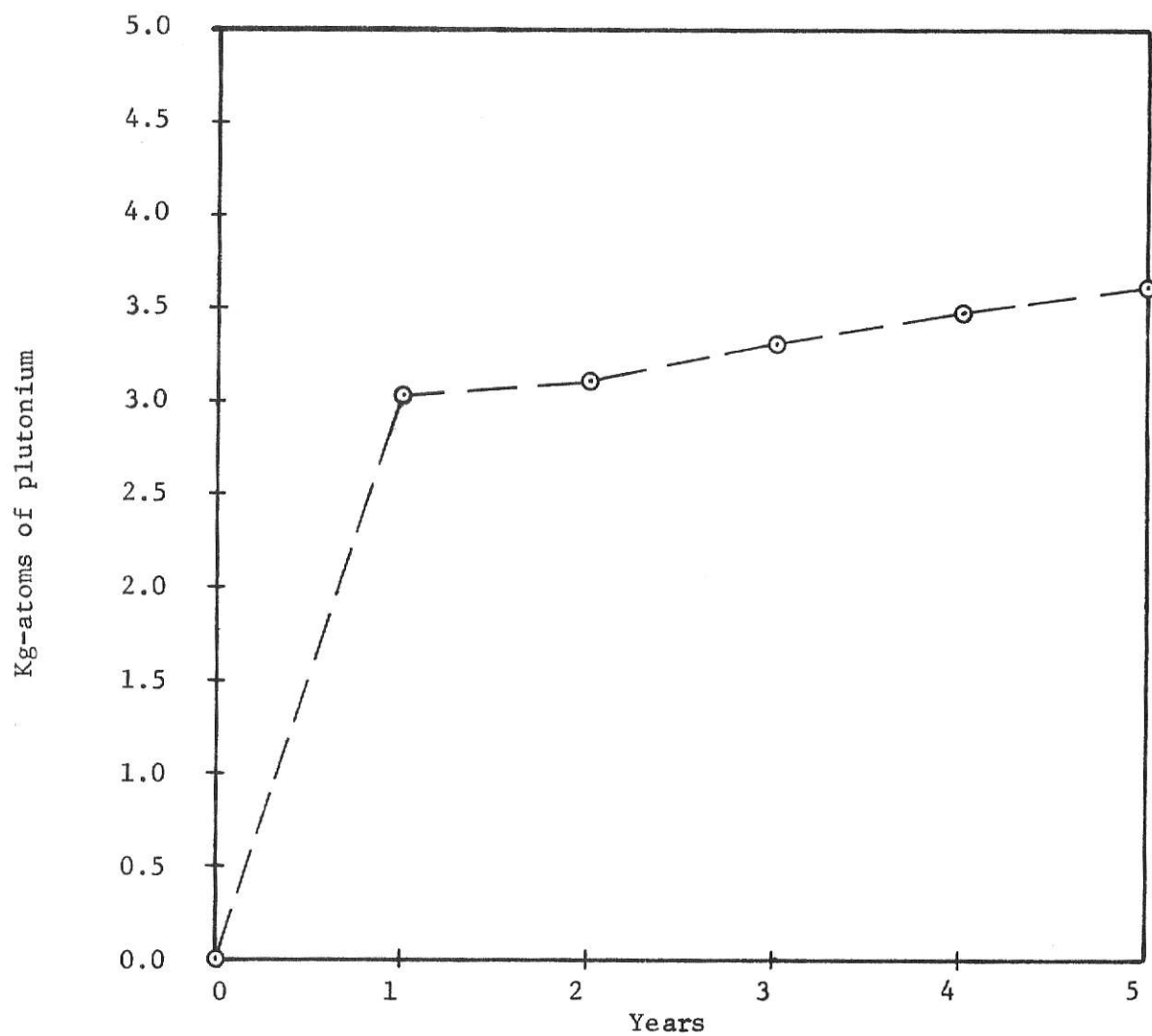


Figure 4.3.17 The amount of fissionable plutonium in the reactor during the fuel cycle vs the number of irradiation periods for the 100% Pu recycle case.

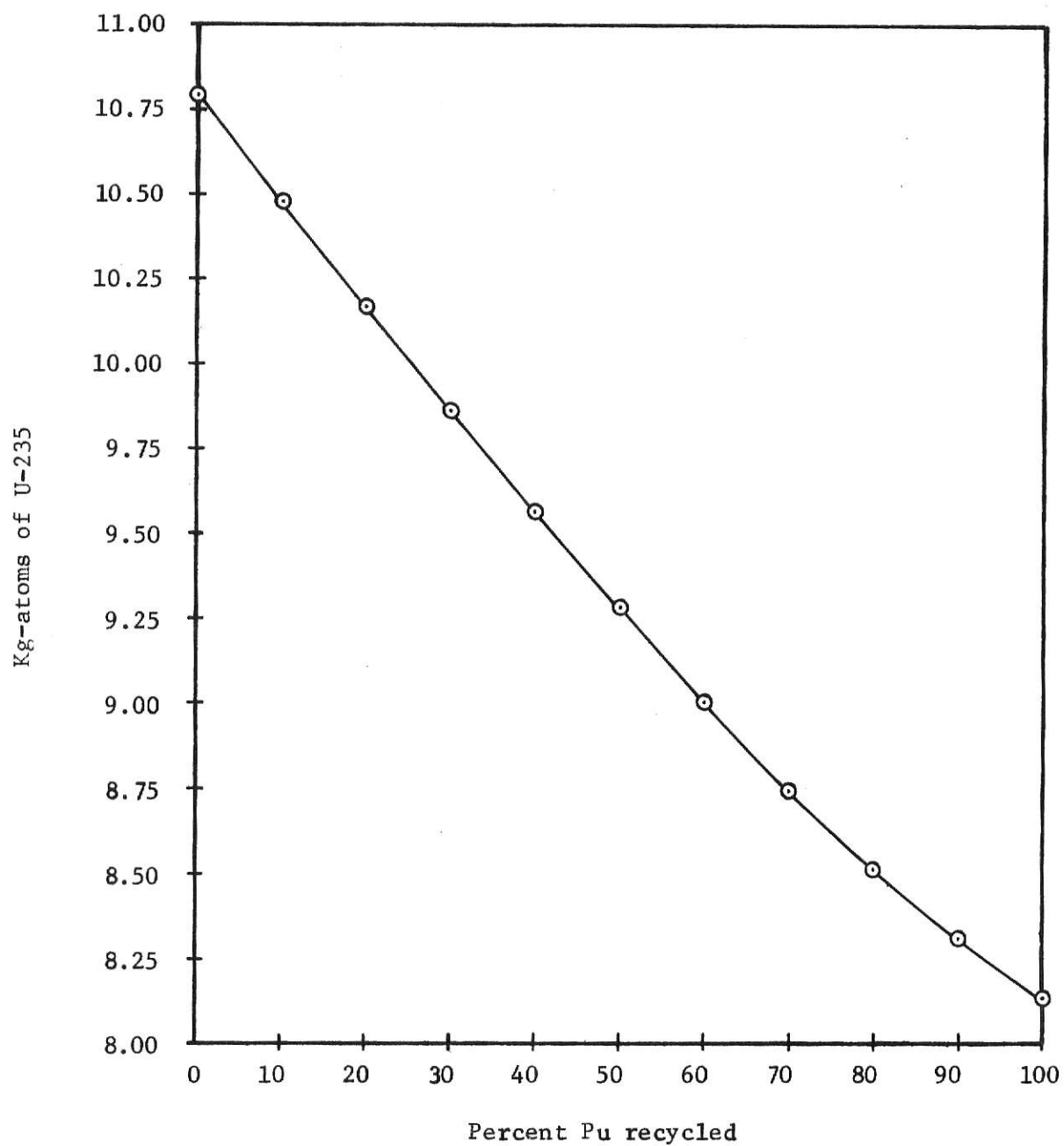


Figure 4.3.18 The value of the objective function vs the percent of Pu recycled.

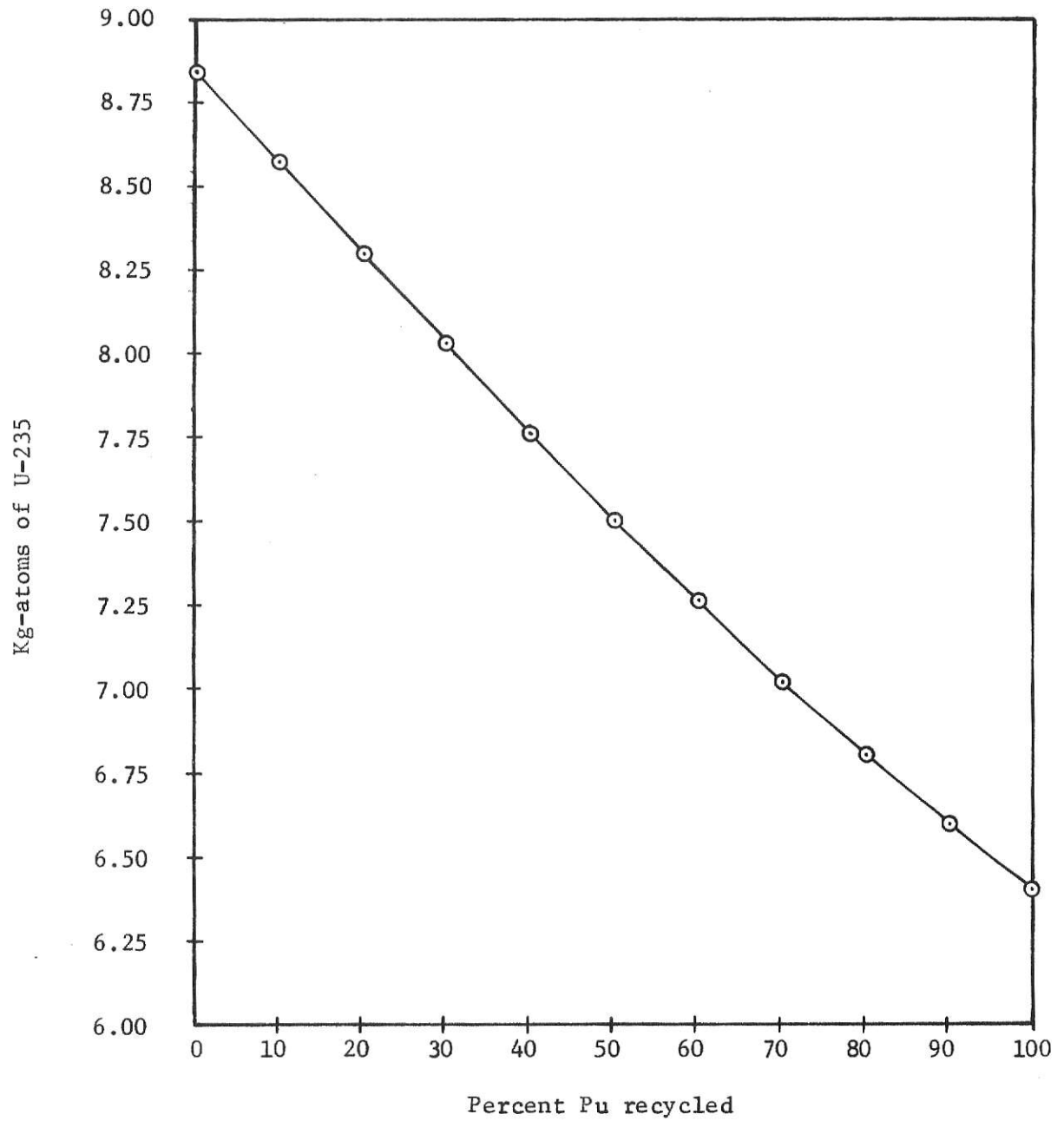


Figure 4.3.19 The amount of U-235 added to the reactor during the fuel cycle vs the percent of Pu recycled.

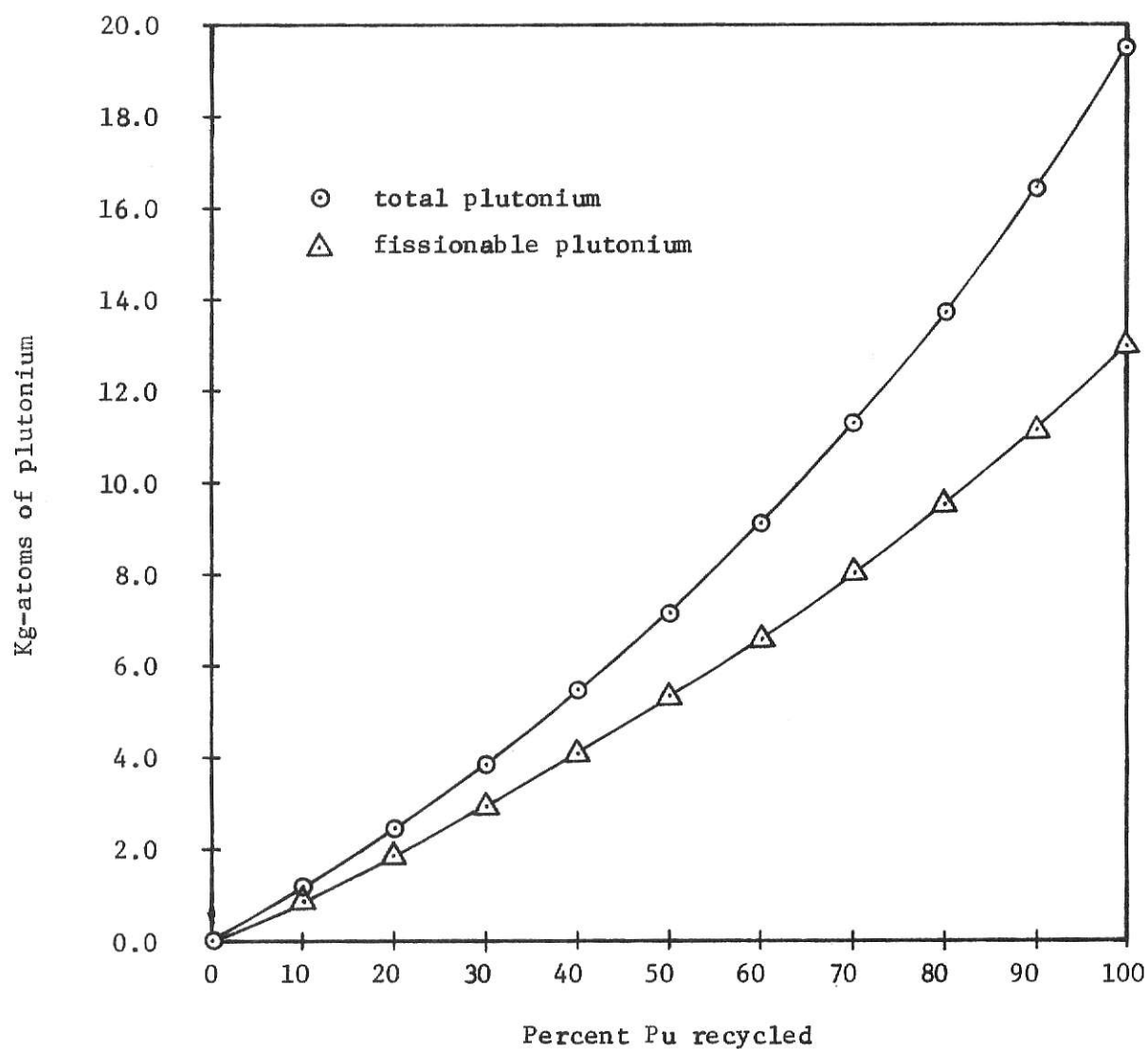


Figure 4.3.20 The amount of plutonium added to the reactor during the fuel cycle vs the percent fo Pu recycled.

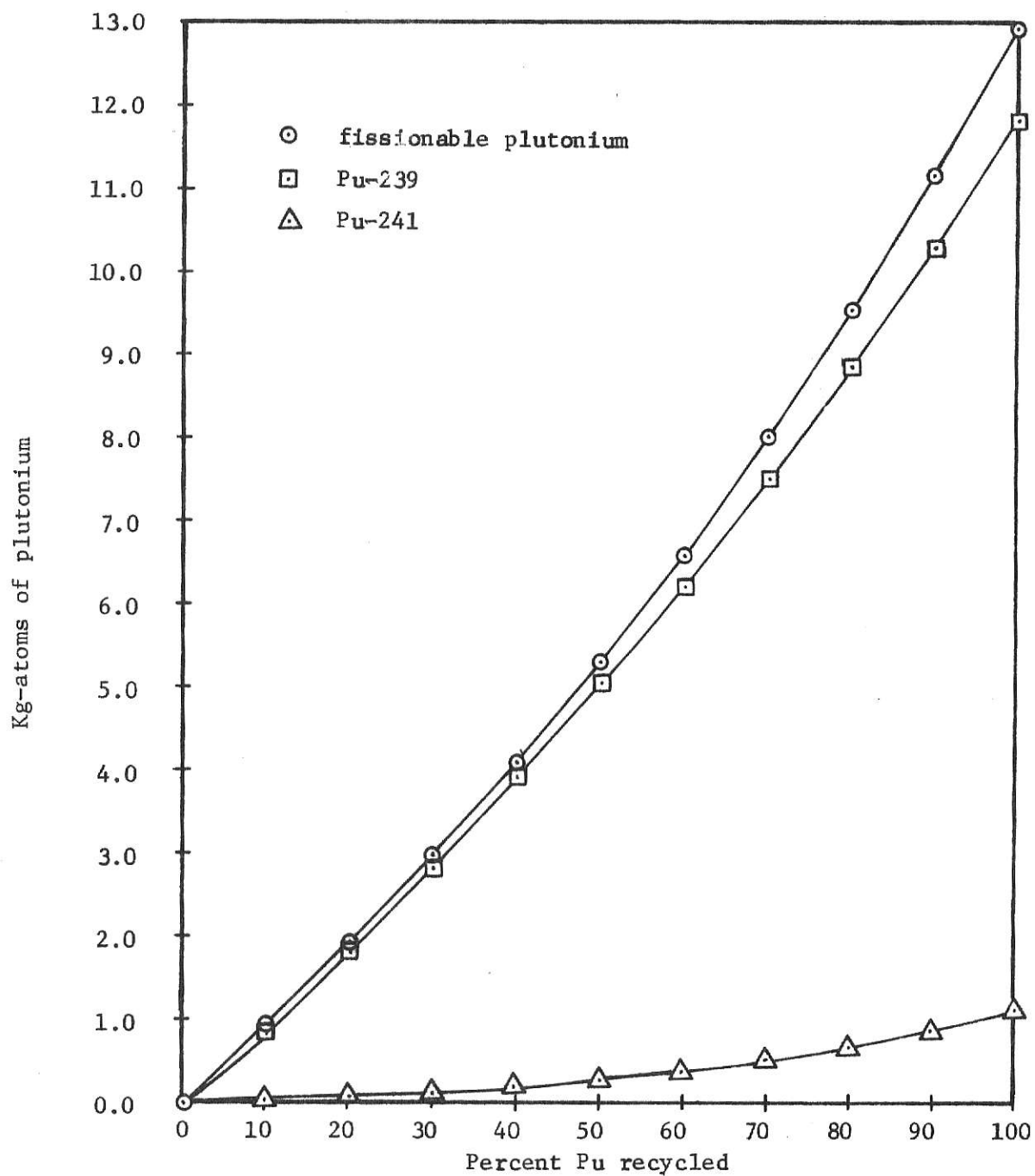


Figure 4.3.21 The amount of fissionable plutonium added to the reactor during the fuel cycle vs the percent of Pu recycled.

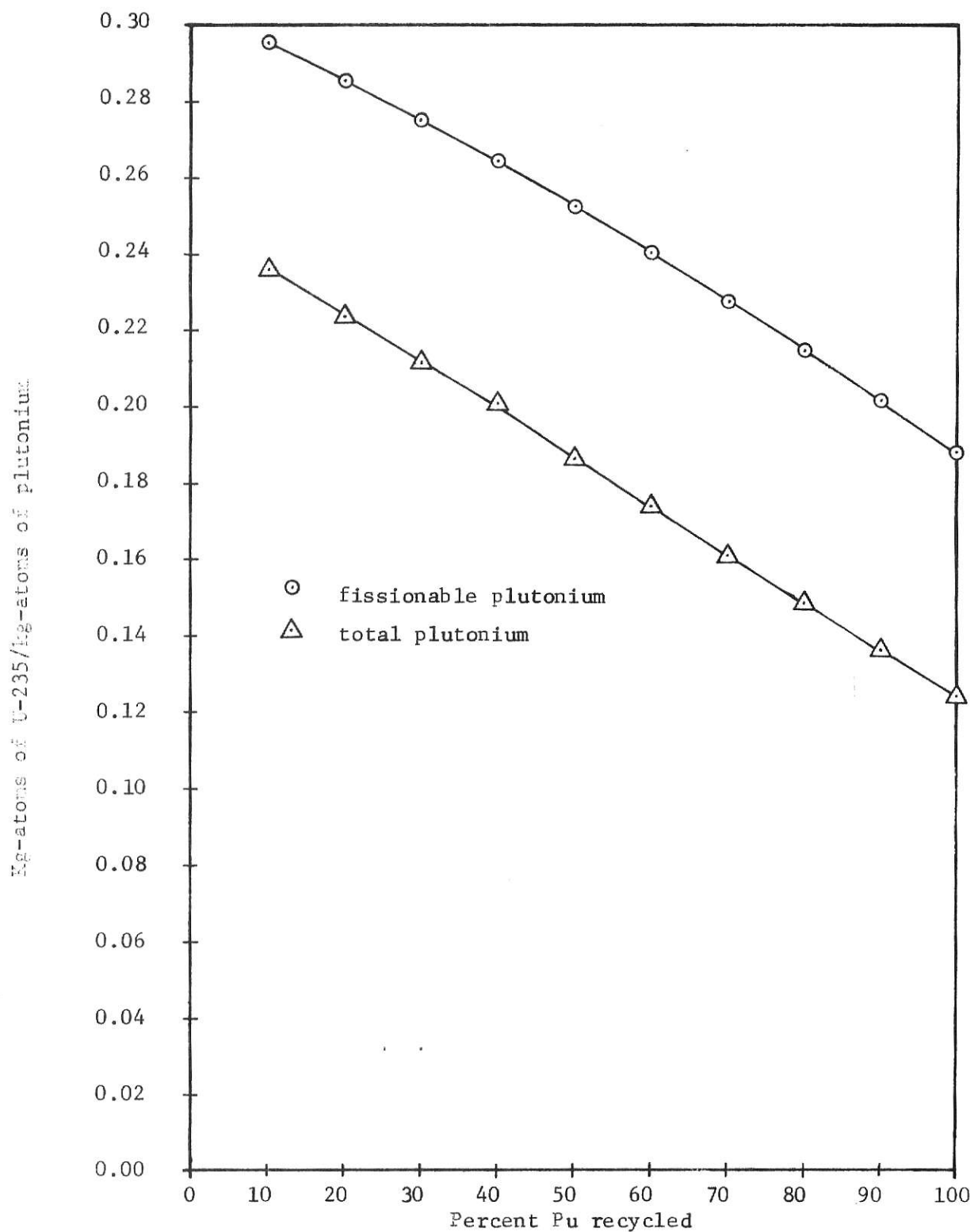


Figure 4.3.22 The ratio of change in the U-235 inventory to change in the Pu inventory vs the percent of Pu recycled.

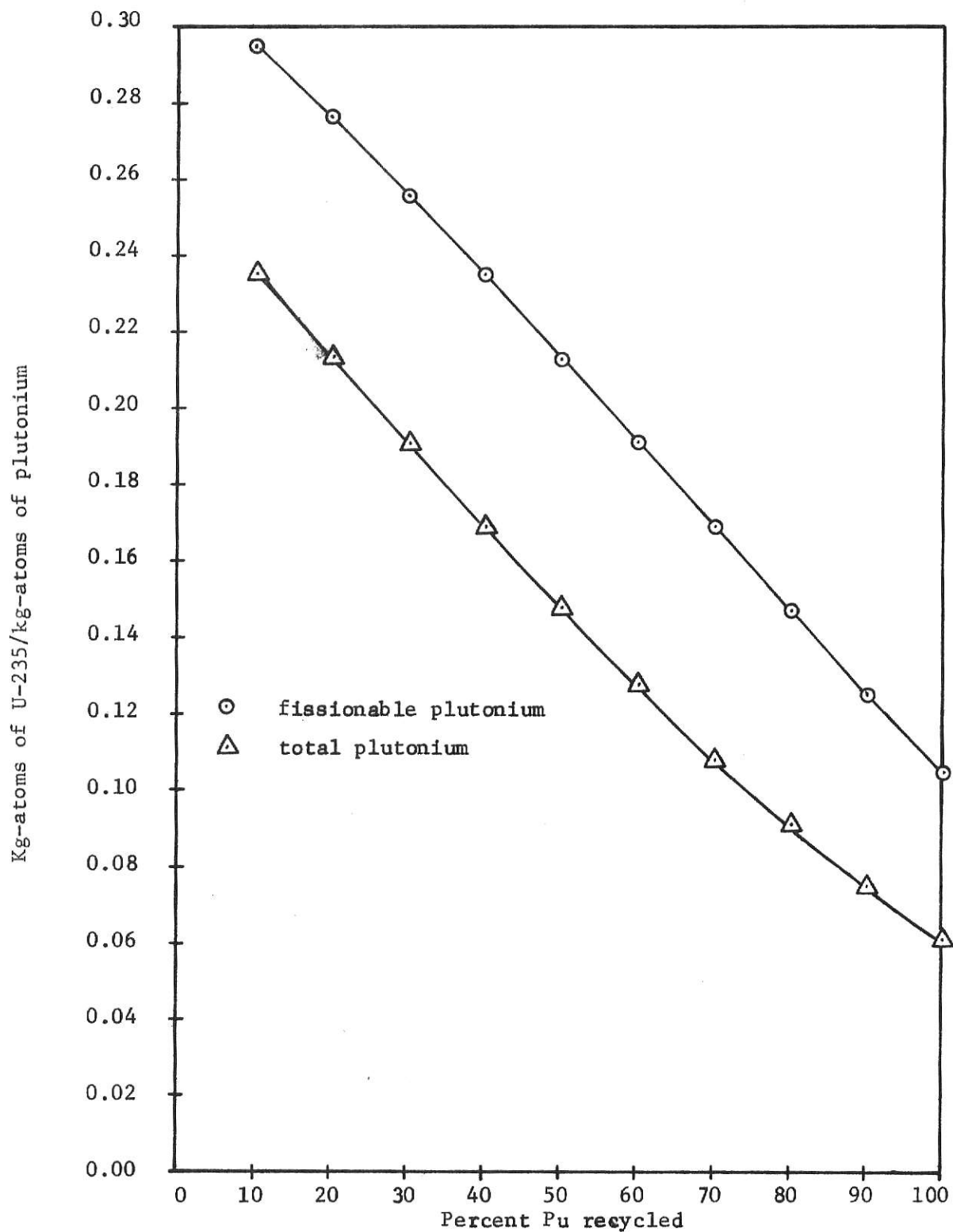


Figure 4.3.23 The ratio of change in the U-235 inventory to the change in the Pu inventory vs the percent of Pu recycled.

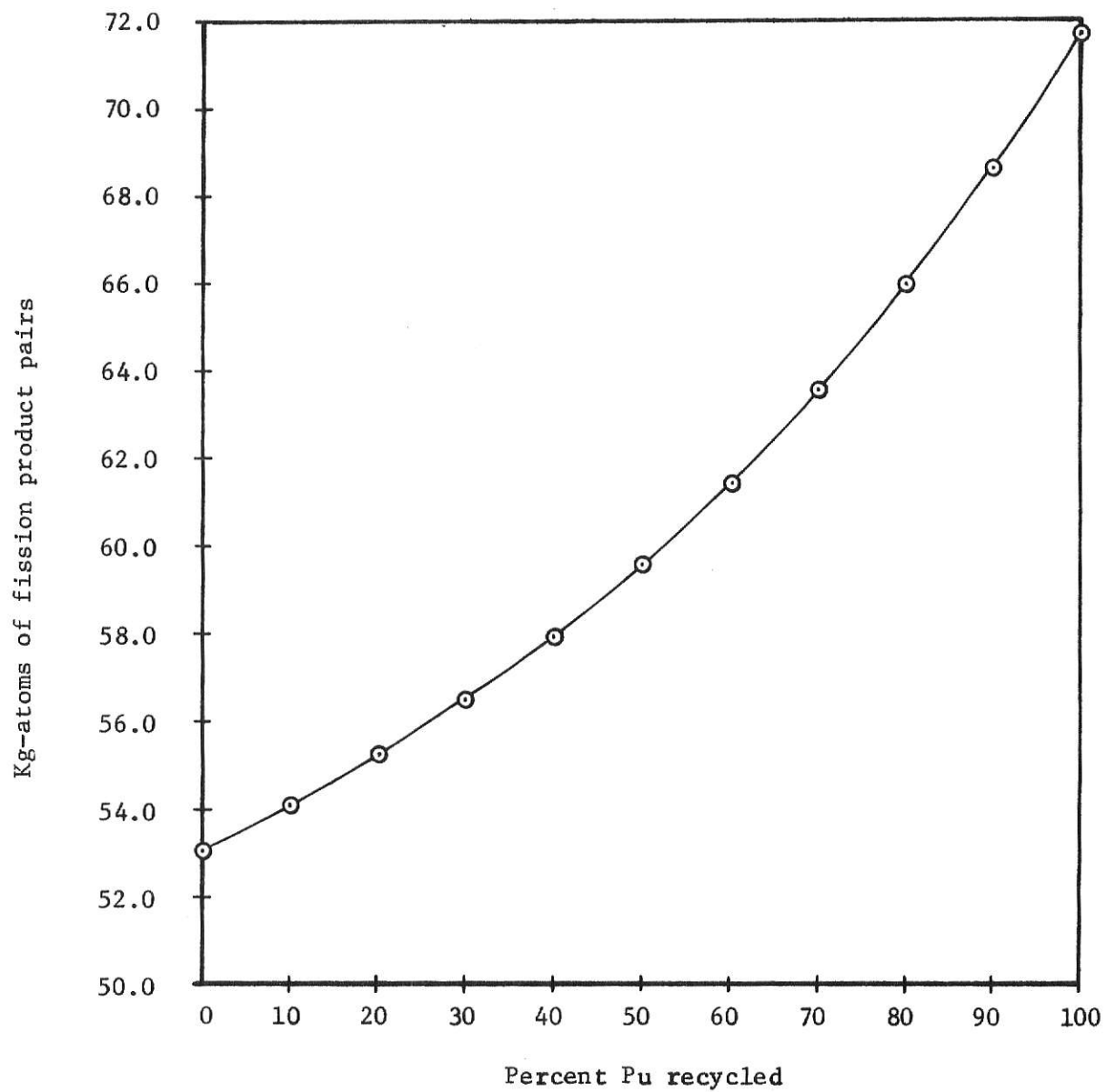


Figure 4.3.24 The amount of fission product pairs removed from the reactor during the fuel cycle vs the percent of Pu recycled.

4.4 Conclusions

As shown in Figs. 4.3.1, 4.3.2, and 4.3.3 the amount of U-235 in the reactor at the start of the second, third, fourth, and fifth irradiation periods is approximately half of the initial loading of U-235. An excess of reactivity is mandatory in the initial loading of a reactor to compensate for build up of fission products and temperature increases, which accounts for the large initial loading of U-235. Also, a result of the assumption of the complete reprocessing of the fuel after each irradiation period is that all fission product pairs are removed. Thus, the amount of U-235 added to the reactor during refueling will be less than the amount of U-235 required when only a fraction of the core is processed. This is particularly evident for refueling at the end of irradiation period one.

The nuclides U-236, Pu-239, Pu-240, Pu-241, and the fission product pairs are dependent on U-235, as shown in Section 3.2, and show a marked increase in production during the first irradiation period. This increase is attributed to the large value of U-235 during that irradiation period. U-238 which is independent of U-235 is used at a constant rate as shown in Fig. 4.3.5. It is also noted that no U-238 is added to the reactor, which is allowed because there is no constraint of minimum enrichment for uranium added.

Figures 4.3.12 through 4.3.17 show the build up of plutonium in the reactor for different percent Pu recycle cases. These figures illustrate the decrease in the removal of plutonium and the increase in the build up of plutonium as the percent Pu recycle increases.

From Fig. 4.3.18 it can be observed that the value of the objective function, i.e., the U-235 requirements, decreases from 10.78938 kg-atoms for 0% Pu recycle to 8.12989 kg-atoms for 100% Pu recycle. The amount of U-235 being added to the reactor during the fuel cycle also decreases from

8.84609 kg-atoms for 0% Pu recycle to 6.40865 kg-atoms for 100% Pu recycle. A 3% minimum enrichment constraint for 0% Pu recycle yields a value for the objective function of 20.05444 kg-atoms and a value of 16.89862 kg-atoms for the amount of U-235 added; almost double the values given for no minimum enrichment constraint. The minimum enrichment of 3% is evidently much larger than is needed to maintain the reactor operation under the assumptions used for this reactor model.

As illustrated in Fig. 4.3.20 the amount of plutonium added to the reactor during the fuel cycle increases from 0 kg-atoms for 0% Pu recycle to 19.51697 kg-atoms for 100% Pu recycle, while the amount of fissionable plutonium added to the reactor increases from 0 kg-atoms for 0% Pu recycle to 12.94503 kg-atoms for 100% Pu recycle. (All of this plutonium is produced by the reactor.) Figure 4.3.21 shows the individual contributions of Pu-239 and Pu-241 to the fissionable plutonium curve. For 100% Pu recycle there are 1.12469 kg-atoms of Pu-241 and 11.82034 kg-atoms of Pu-239 of which 11.81882 kg-atoms are produced from thermal absorption in U-238 and .00152 kg-atoms are produced from thermal absorption in U-235. In every case considered, the majority of fissionable plutonium produced was Pu-239. More than 99.9% of this Pu-239 was produced from thermal absorption in U-238.

Pu-239 increases in a non linear fashion because of the I_{98}^0 , i.e., the amount of Pu-239 resulting from thermal absorption in U-238 in the previous irradiation period, term in Eq. 3.2.16. I_{98}^0 will vary depending on the percent of Pu recycle. The other plutonium isotopes considered have a similar dependency on the value of the isotope in the previous period. The constraints of the model allow for plutonium to be used as a fissile material, thus reducing the demand for U-235. Therefore, the amount of plutonium increases

non-linearly. Hence, the amount of U-235 decreases non-linearly. A plot of the ratio of change in the U-235 inventory to the change in the plutonium inventory versus the percent of Pu recycled amplifies these non-linearities. Figures 4.3.22 and 4.3.23 illustrate these results. In Fig. 4.3.22 the ratio decreases from .29536 kg-atom U-235/ kg-atom fissionable plutonium for 10% Pu recycle to .18829 kg-atom U-235/ kg-atom fissionable plutonium for 100% Pu recycle. Also in Fig. 4.3.22, when the total amount of plutonium is considered, the ratio decreases from .23615 kg-atom U-235/ kg-atom plutonium for 10% Pu recycle to .12489 kg-atom U-235/ kg-atom plutonium for 100% Pu recycle. In Fig. 4.3.23 the curves have a greater rate of decrease: from .29536 kg-atom U-235/ kg-atom fissionable plutonium to .10408 kg-atom U-235/ kg-atom fissionable plutonium and from .23615 kg-atom U-235/ kg-atom plutonium to .06057 kg-atom U-235/ kg-atom plutonium for 10% and 100% Pu recycle, respectively. The negative slope of these curves is attributed to the fact that the U-235 inventory decreases while the plutonium inventory increases, as shown in Figs. 4.3.19 and 4.3.20. The increasing negative slope of the change in the required U-235 ratioed to the change in plutonium used shown in Fig. 4.3.23 and is explained as follows. The rate of decrease in the U-235 inventory is greater as the percent of Pu recycle increases. Also, the rate of increase in the plutonium inventory is greater as the percent of Pu recycle increases.

This ratio is important in relating the value of plutonium to the cost of U-235. The value of U-235 is a fixed and known quantity, while the value of plutonium has not been firmly established. Figures 4.3.22 and 4.3.23 illustrate the result that the value of plutonium is a function of the percent of Pu recycle considered. Voila: Its value decreases as the percent of Pu recycle increases.

In summary, the results presented above indicate that this model is a fair representation of a power reactor refueling scheme. It can be made more complex and accurate by reevaluating the assumptions and adding more constraints. This model is quite versatile in the manner in which many different studies can be made of the same reactor. For example, by changing the objective function, it is possible to maximize the amount of plutonium removed, or to maximize the plutonium removed while minimizing the U-235 requirements. Thus, the model's versatility and availability make it an attractive addition to the already existing refueling codes.

5.0 SUGGESTIONS FOR FURTHER STUDY

Further study could be completed on this model by comparing different refueling schemes, such as comparing a four thirds or five fourths core refueling scheme with an unspecified fractional core refueling philosophy (7). Another study could be to divide the core into three or more zones, thus a flux profile could be simulated in the reactor. This model could be combined with a fuel element shuffling technique in such a manner as to resemble the actual refueling process (8). Another study may be to compare the results of the burnup code used in this work to more complex codes such as FEVER (2).

Other studies may be completed by including a fuel enrichment constraint for the addition of uranium to the reactor, or by considering the reactivity holdddown requirements for a plutonium recycle reactor (1).

6.0 ACKNOWLEDGEMENTS

I wish to express my gratitude to Dr. N. Dean Eckhoff whose encouragement and direction made this work possible. Also, the financial support of the Department of Nuclear Engineering, and the National Science Foundation Traineeship program is greatly appreciated. A note of thanks is extended to Dr. R. E. Faw, Professor, Department of Nuclear Engineering, and Dr. L. E. Grosh, Professor, Department of Industrial Engineering for their suggested improvements.

A special thank you is extended to Maureen Beaudet for her assistance in typing this manuscript, and Donald Prigel for his assistance in drawing the figures. Finally, I wish to express my love and appreciation to my wife, Linda, for the long hours spent typing this thesis and her continued support and encouragement.

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8.0 APPENDICES

APPENDIX A

A.1 Nuclide Burnup Code

This code was written so that the constants for the burnup and production equations listed in Section 3.2 could be calculated using the assumptions stated in Section 3.2. The code is similar to one written by Hawk (7), with the exception that this code includes the nuclides Np-237, Pu-238, and Pu-239 produced from thermal absorption in U-235. The input data are included in the code and comprize the initial amounts of each nuclide, the absorption, fission, and capture cross sections for each nuclide, and ϵ , P_1 , P , η_5 , η_9 , θ .

The code is written for the IBM 360 Model 50 computer and requires approximately .23 seconds execution time.

A.2 Nuclide Burnup Code Listing

**THIS BOOK WAS
BOUND WITH
MULTIPLE PAGES
THAT HAVE PRINTING
EXTENDING INTO THE
BINDING, CAUSING
THE PRINTING TO BE
CUT OFF.**

**THIS IS AS RECEIVED
FROM THE
CUSTOMER.**

```

1 Y05= 10.6383
2 Y06= .00
3 Y08= 297.5375
4 Y09= .00
5 Y095= 0.0
6 Y00= .00
7 Y01= .00
8 Y07= .00
9 Y0N7= 0.0
0 Y0P8= 0.0
1 ANU5=2.47
2 ANU8=2.55
3 ANU9=2.905
4 ANU1=3.060
5 SIGS= .25148
6 SA5= .678
7 SA6= .0055
8 SA8= .00273
9 SA9= 1.013
0 SA0= .280
1 SA1= 1.364
2 SAF= .0319
3 SAP8= .540
4 SAN7= .169
5 SF5= .580
6 SF8= .000
7 SF9= .742
8 SF1= 1.0
9 SC5= .098
0 SC8= .00273
1 SC9= .271
2 SC0= .280
3 SCN7= .169
4 SC6= .0055
5 SCP8= .540
6 EPS = 1.0584
7 P1 = .9742
8 P = .758
9 ETA5 = 1.97
0 ETA9 = 1.81
1 T= 1.5552
2 R11=EXP(-SA5*T)
3 R21=SC5/((SA6-SA5)*(EXP(-SA5*T)-EXP(-SA6*T)))
4 R22=EXP(-SA6*T)
5 R33=1.0
6 C9 = EPS*P1*(1.0-P)*SA9*ETA9
7 C5 = EPS*P1*(1.0-P)*SA5*ETA5
8 C91 = C9 - SA9
9 R41=(C5/((SA5+C91))*(EXP(C91*T)-EXP(-SA5*T)))
0 R44=EXP(C91*T)
1 C8=-SC6/C91
2 R40=C8*(1.0-EXP(C91*T))
3 A=(C91+SA0)
4 B=(SA5+C91)
5 C=(SA5-SA0)
6 R51=(C5*SC9/((A*B))*(EXP(C91*T)-EXP(-SA0*T)))+(C5*SC9/((B*C))
7 1*(EXP(-SA5*T)-EXP(-SA0*T)))
8 R54=SC9*(EXP(C91*T)-EXP(-SA0*T))/A
9 R55=EXP(-SA0*T)

```

```

R50=C8*SC9*(1.0-EXP(-SA0*T))/SA0+C8*SC9*(EXP(-SA0*T)-EXP(C91*T))/A
D=(SA1-SA0)
E=(C91+SA1)
F=(SA1-SA5)
R61=SCC*SC9*C5*(EXP(C91*T)/(E*A*B)+EXP(-SA5*T)/(F*C*B))
1-(SC0*SC9*C5*EXP(-SA0*T)/(D*B))*(1.0/A+1.0/C)-(SC0*SC9*C5
2*EXP(-SA1*T)/B)*(1.0/(E*A)+1.0/(F*C)-1.0/(D*A)-1.0/(D*C))
R64=SCC*SC9*EXP(C91*T)/(E*A)-SC0*SC9*EXP(-SA0*T)/(D*A)
1-(SC0*SC9*EXP(-SA1*T)/A)*(1.0/E-1.0/D)
R65=(SC0/D)*(EXP(-SA0*T)-EXP(-SA1*T))
R66=EXP(-SA1*T)
R60=SC0*C8*SC9*(1.0-EXP(-SA1*T))/(SA1*SA0)-SC0*C8*SC9*(EXP(C91*T)
1-EXP(-SA1*T))/(A*E)+SC0*C8*SC9*(EXP(-SA0*T)-EXP(-SA1*T))*(1.0/(A*
2D)-1.0/(SA0*D))
R71=2.0*C5*(EXP(C91*T)-1.0)/(C91*B)*(SF9+SF1*SC0*SC9/(E*A))
1+(2.0*(EXP(-SA5*T)-1.0)/SA5)*(SF9*C5/B-SF5-SF1*SC0*SC9*C5
2/(F*C*B))+(2.0*SF1*SC0*SC9*C5*(EXP(-SA0*T)-1.0)/(SA0*D*B))*(1.0
3/C+1.0/A)+(2.0*SF1*C5*SC0*SC9*(EXP(-SA1*T)-1.0)/(SA1*B))
4*(1.0/(A*E)+1.0/(F*C))-((2.0*SF1*SC0*SC9*C5*(EXP(-SA1*T)-1.0)/
5(SA1*B*D))*(1.0/A+1.0/C)
R74=(2.0*SF9*(EXP(C91*T)-1.0)/C91)*(1.0+SC0*SC9/(A*E))
1+2.0*SF1*SC0*SC9*(EXP(-SA0*T)-1.0)/(SA0*D*A)+(2.0*SF1*SC0*SC9*
2(EXP(-SA1*T)-1.0)/(SA1*A))*(1.0/E-1.0/D)
R75=2.0*SF1*SC0*(EXP(-SA1*T)-1.0)/(SA1*D)-2.0*SF1*SC0*
1(EXP(-SA0*T)-1.0)/(SA0*D)
R76=-2.0*SF1*(EXP(-SA1*T)-1.0)/SA1
BB=(EXP(-SA0*T)-1.0)
CC=(EXP(-SA1*T)-1.0)
R70=2.0*C8*(SF9+SF1*SC0*SC9/(SA0*SA1))*T-2.0*C8*(EXP(C91*T)-1.0)
1*(SF9/C91+SF1*SC0*SC9/(C91*A*E))+((2.0*SF1*SC0*SC9*C8/(SA0*D))
2*(1.0/SA0-1.0/A)*BB+(2.0*SF1*C8*SC0*SC9/SA1)*(1.0/(SA1*SA0)
3-1.0/(A*E))*CC+(2.0*SF1*SC0*SC9*C8/(SA1*D))*(1.0/A-1.0/SA0)*CC
DD=EXP(-SA5*T)-EXP(-SCN7*T)
EE=EXP(-SCN7*T)-EXP(-SA6*T)
G=SCN7-SA5
Q=SCN7-SA6
V=SA6-SA5
R91=SC6*SC5*(DD/G+EE/Q)/(SA6-SA5)
R92=SC6*(-EE)/Q
R93=EXP(-SCN7*T)
FF=EXP(-SA5*T)-EXP(-SCP8*T)
GG=EXP(-SA6*T)-EXP(-SCP8*T)
QQ=EXP(-SCN7*T)-EXP(-SCP8*T)
R=SCP8-SA5
S=SCP8-SA6
U=SCP8-SCN7
R81=SCN7*SC6*SC5*FF/(R*G*V)-SCN7*SC6*SC5*GG/(S*Q*V)
1+SCN7*SC6*SC5*QQ*(1.0/Q-1.0/G)/(U*V)
R82=SCN7*SC6*(GG/S-QQ/U)/Q
R83=SCN7*QQ/U
R84=EXP(-SCP8*T)
RR=EXP(-SA5*T)-EXP(-SA9*T)
SS=EXP(-SA6*T)-EXP(-SA9*T)
UU=EXP(-SCN7*T)-EXP(-SA9*T)
VV=EXP(-SCP8*T)-EXP(-SA9*T)
W=SA5-SA9
Z=SA6-SA9
WW=SCN7-SA9
ZZ=SCP8-SA9
R101=SCP8*SCN7*SC6*SC5*(SS/(Z*S*Q*V)-RR/(W*R*G*V))

```

```

1+SCP8*SCN7*SC6*SC5*UU*(1.0/G-1.0/G)/(WW*U*V)+SCN7*SCP8*SC6*SC5*
2VV*(1.0/(R*G)-1.0/(S*Q)+1.0/(U*Q)-1.0/(U*G))/(ZZ*V)
R102=SCP8*SCN7*SC6*(UU/(WW*U)-SS/(Z*S))/Q+SCP8*SCN7*SC6*(1.0/S- 70
11.0/U)*VV/(ZZ*Q)
R103=SCP8*SCN7*(VV/ZZ-UU/WW)/U
R104=-SCP8*VV/ZZ
R105=EXP(-SA9*T)
Y5=R11*Y05
Y6=R21*Y05+R22*Y06
Y8=R33*Y08
Y9=R41*Y05+R44*Y09+R40*Y08
Y95=R101*Y05 +R102*Y06+R103*Y0N7+R104*Y0P8+R105*Y095
Y0=R51*Y05+R54*Y09+R55*Y00+R50*Y08
Y1=R61*Y05+R64*Y09+R65*Y00+R66*Y01+R60*Y08
YN7=R91*Y05+R92*Y06+R93*Y0N7
YP8=R81*Y05+R82*Y06+R83*Y0N7+R84*Y0P8
Y7=R71*Y05+R74*Y09+R75*Y00+R76*Y01+R70*Y08
S1=ANU5*SF5*Y5+ANU8*SF8*Y8+ANU9*SF9*(Y9+Y95)+ANU1*SF1*Y1
S2=SA5*Y5+SA6*Y6+SA8*Y8+SA9*(Y9+Y95)+SA0*Y0+SA1*Y1+SAF*Y7
1+SAN7*YN7+SAP8*YP8
FK=S1/(S2+SIGS)
WRITE(6,20) R11,R21,R22,R33,R41,R44,R40,R51,R54,R55,R50,R61,R64,
1R65,R66,R60,R71,R74,R75,R76,R70,T,FK
20 FORMAT(1X,4E15.8,/1X,4E15.8,/1X,4E15.8,/1X,4E15.8,/1X,4E15.8,
1/1X,3E15.8)
WRITE(6,21) R91,R92,R93,R81,R82,R83,R84,R101,R102,R103,R104,R105
21 FORMAT(1X,4E15.8,/1X,4E15.8,/1X,4E15.8)
STOP
END

```

\$ENTRY

```

0.34839260E 00 0.93714230E-01 0.99148280E 00 0.10000000E 01
0.19892650E 00 0.42151620E 00 0.28430070E-02 0.50264770E-01
0.22177940E 00 0.64696960E 00 0.58927640E-03 0.53725830E-02
0.31170630E-01 0.13614940E 00 0.11987660E 00 0.58577620E-04
0.14443610E 01 0.17714070E 01 0.31800720E 00 0.12905020E 01
0.38007550E-02 0.15551990E 01 0.16089070E 01
0.42914150E-03 0.74883840E-02 0.76887350E 00 0.33830750E-04
0.78768790E-03 0.15354850E 00 0.43179340E 00 0.58242580E-05
0.16650740E-03 0.46836150E-01 0.25672500E 00 0.20692150E 00

```

APPENDIX B

B.1 MPS Computer Program

This program uses the revised simplex method to solve LP problems. The following two pages (9,10), obtained from Dr. Said Ashour (11) in his Linear Programming course during the spring semester 1971, describe the control cards on page 1 of the printout. The input data include a list of the rows, columns, and RHS (right hand side) of the matrix in Fig. 3.4.1, and are printed in the format shown on pages 5-12 of the printout. This printout is for the 50% Pu recycle case.

Short Notes On MPS Computer Program

PROGRAM. PROGRAM is a mandatory statement at the beginning of each program.

INITIAL Z. INITIAL Z is used to establish initial settings for tolerance, frequencies and demands. This is a system macro.

TITLE ('EXAMPLE'). This statement is for giving a suitable name to the problem.

MOVE (XDATA, 'EXAMPLE') and MOVE (XPBNAME, 'PBFILE'). The first statement moves input data set name EXAMPLE into the cell XDATA, and the second moves the problem file name, PBFILE, into the cell XPBNAME.

CONVERT. This routine is the master of the convert procedure, which is used to convert external input data into a binary model and transfers this to PBFILE.

SETUP ('MAX'). This routine is the master of the setup procedure, which set up the problem name in XPBNAME by

1. Searching for the problem
2. Opening the matrix, etc, and scratch files
3. Making the storage allocation
4. Building the work matrix
5. Setting the logical basis and structural bounded variables at upper bound
6. Building the inverse of the logical basis

For minimization 'MAX' should be changed to 'MIN'.

BCDOUT. This converts the specified binary problem into the external input data format. The output may be listed and/or punched out and is in the order of the input data sections, ROWS, COLUMNS, RHSS, RANGES, AND BOUNDS. The NAME and ENDATA cards may also be produced.

MOVE (XOBJ, 'PROFIT') and MOVE (XRHS, 'LIMITS'). These statements are for identifying the objective function and the right hand side limits. For minimizing 'PROFIT' becomes 'COST'. Objective function and right hand side have to be named because a program can work with many objective functions and limits.

PICTURE. PICTURE procedure produces a "picture" of the current work matrix in condensed format. 45 rows and up to 55 columns for an output page are given. The pages are numbered using matrix notation for ease of identification. The output of PICTURE gives a quick visual check to see whether the structure of the matrix is correct and whether any coefficients are missing.

The range on the problem right hand side, and bounds on variables are indicated if they exist.

PRIMAL. PRIMAL optimizes the current problem using a composite primal algorithm. It can work with a composite objective function and/or a composite right hand side. It can work with a composite restraint row if neither the objective function nor the right hand side is composite.

SOLUTION. The output of a summary of the solution corresponding to the current basis. This output may be either printed or filed depending on a parameter in the procedure card.

EXIT. EXIT is a procedure linked to by the executor before return to the operating system. It is mainly used to close data sets and print the time of executor job step.

PEND. This is equivalent to END statement in FORTRAN PROGRAM.

B.2 MPS Computer Program Listing

C001	PROGRAM
C002	INITIALZ
C064	MOVE(XDATA,'CHART')
C065	MOVE(XPBNAME,'PBFILE')
C066	CONVERT('SUMMARY')
C067	SETUP
C068	BCDOUT
C069	MOVE(XOBJ,'OBJ')
C070	MOVE(XRHS,'RHS')
C071	PICTURE
C072	PRIMAL
C073	SOLUTION
C074	EXIT
C075	DC(0.0)
C076	DC(0.0)
C077	PEND

SV1
SV2

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CONVERT CHART TC PBFILE

TIME = 0.06

SUMMARY

1- ROWS SECTION.

0 MINOR ERROR(S) - 0 MAJOR ERROR(S).

2- COLUMNS SECTION.

0 MINOR ERROR(S) - 0 MAJOR ERROR(S).

3- RHS'S SECTION.

RHS

0 MINOR ERROR(S) - 0 MAJOR ERROR(S).

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NUMBER OF ELEMENTS BY COLUMN ORDER

112	Y05	11	Y06	6	Y08	7	Y098	6	Y095	6	Y00	5	Y01
119	Y0N7	5	Y0P3	4	Y07	2	Y15	14	Y16	8	Y18	10	Y196
126	Y195	9	Y10	8	Y11	7	Y1N7	8	Y1P6	7	Y17	9	Y25
133	Y26	8	Y28	10	Y298	9	Y295	9	Y20	8	Y21	7	Y2V7
140	Y2P8	7	Y27	5	Y35	14	Y36	8	Y38	10	Y39C	9	Y395
147	Y30	8	Y31	7	Y3N7	8	Y3P8	7	Y37	5	Y45	14	Y46
154	Y48	10	Y498	9	Y495	9	Y40	8	Y41	7	Y47	6	Y4P6
161	Y47	5	Y55	3	Y56	2	Y58	2	Y598	2	Y595	2	Y50
168	Y51	2	Y5N7	2	Y5P8	2	Y57	2	U15	3	U16	3	U19
175	U198	3	U195	3	U10	3	U11	3	U1N7	2	U1P8	3	U17
182	U25	3	U26	3	U28	2	U298	3	U295	3	U20	3	U21
189	U2N7	2	U2P8	3	U27	2	U35	3	U36	3	U38	2	U395
196	U395	3	U30	3	U31	3	U3N7	2	U3P8	3	U37	2	U45
203	U46	3	U48	2	U498	3	U495	3	U40	3	U41	3	U47
210	U4P8	3	U47	2									

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NUMBER OF ELEMENTS BY ROW ORDER, EXCLUDING RHS'S, INCLUDING SLACK ELEMENT

1 N 08J	6	E E05	2	E E06	2	E E08	2	E E098	2	E E099	2	E E00
8 E E01	2	E E0N7	2	E E0P8	2	E E07	2	E E15	2	E E16	2	E E18
15 E E198	6	E E195	8	E E10	8	E E11	8	E E1N7	8	E E1P8	7	E E17
22 E E25	4	E E26	5	E E28	4	E E298	4	E E295	6	E E10	8	E E21
29 E E2N7	6	E E2P8	7	E E27	7	E E35	4	E E36	4	E E38	4	E E398
36 E E395	8	E E30	8	E E31	9	E E3N7	6	E E3P8	6	E E37	7	E E45
43 E E46	5	E E48	4	E E498	7	E E495	7	E E40	7	E E41	9	E E47
50 E E4P8	7	E E47	9	E E55	3	E E56	4	E E58	3	E E598	5	E E595
57 E E50	7	E E51	8	E E5N7	5	E E5P8	6	E E57	6	E E12	11	E E15
64 G C14	11	G C15	11	G C21	21	G C22	21	G C23	21	G C24	11	G C19
71 L C42	3	L C43	3	L C44	3	L C45	3	L V11	3	L V12	11	L V13
78 L V14	11	L V15	11	E C01	3	E C02	3	E C03	3	E C04	3	E C31
85 E C32	3	E C33	3	E C34	3	E C51	3	E C52	3	E C53	3	E C54
92 E C61	2	E C62	2	E C63	2	E C64	2	E C71	2	E C72	3	E C73
99 E C74	3	E C91	2	E C92	2	E C93	2	E C94	2	E C101	2	E C102
106 E C103	2	E C104	2	E C111	3	E C112	3	E C113	3	E C114	3	E C102

PROBLEM STATISTICS - 111 ROWS, 211 VARIABLES, 636 ELEMENTS, DENSITY = 2.71

THESE STATISTICS INCLUDE ONE SLACK VARIABLE FOR EACH ROW.

0 MINOR ERRORS, 0 MAJOR ERRORS.

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SETUP PBFILE

TIME = 0.51

MATRIX1 ASSIGNED TO MATRIX1

ETA1 ASSIGNED TO ETA1

SCRATCH1 ASSIGNED TO SCRATCH1

SCRATCH2 ASSIGNED TO SCRATCH2

MAXIMUM PRICING NOT REQUIRED - MAXIMUM POSSIBLE 7

NO CYCLING

POOLS	NUMBER	SIZE	CORE
H-REG-BITS MAP			456
WORK REGIONS	9	912	8208
MATRIX BUFFERS	4	3504	14016
ETA BUFFERS	4	14304	57216

ROWS (LOG.VAR.)	TOTAL	NORMAL	FREE	FIXED	BOUNCED
COLUMNS (STR.VAR.)	111	18	1	92	0
	100	100	0	0	0

636 ELEMENTS - DENSITY = 2.71 - 3 MATRIX RECORDS (WITHOUT RHS'S)

BCDCUT - USING PBFILE

TIME = 0.57

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CHART

NAME
RCWS

N 0BJ
E E05
E E06
E E08
E E098
E E095
E E00
E E01
E E0N7
E E0P8
E E07
E E15
E E16
E E18
E E198
E E195
E E10
E E11
E E1N7
E E1P8
E E17
E E25
E E26
E E28
E E298
E E295
E E20
E E21
E E2N7
E E2P8
E E27
E E35
E E36
E E38
E E398
E E395
E E30
E E31
E E3N7
E E3P8
E E37
E E45
E E46
E E48
E E498
E E495
E E40
E E41
E E4N7
E E4P8
E E47
E E55

E E56
E E58
E E598
E E595
E E50
E E51
E E5N7
E E5P8
E E57
G C12
G C13
G C14
G C15
G C21
G C22
G C23
G C24
G C25
L C42
L C43
L C44
L C45
L V11
L V12
L V13
L V14
L V15
E C01
E C02
E C03
E C04
E C31
E C32
E C33
E C34
E C51
E C52
E C53
E C54
E C61
E C62
E C63
E C64
E C71
E C72
E C73
E C74
E C91
E C92
E C93
E C94
E C101
E C102
E C103

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EXECUTCR. MPS/360 V2-M8

E C104
E C111
E C112
E C113
E C114
COLUMNS

E05	1.00000	E15	.34839
E16	.09371	E198	.19893
E195	.00001	E10	.05026
E11	.00537	E1N7	.00043
E1P8	.00003	E17	1.44436
V11	12.40100		
E06	1.00000	E16	.99148
E195	.00017	E1N7	.00749
E1P8	.00079	V11	12.43400
E08	1.00000	E18	.99576
E198	.00284	E10	.00059
E11	.00006	E17	.00380
V11	12.55900		
E098	1.00000	E198	.42152
E10	.22178	E11	.03117
E17	1.77141	V11	12.04600
E095	1.00000	E195	.20692
E10	.22178	E11	.03117
E17	1.77141	V11	12.04600
E00	1.00000	E10	.64697
E11	.13615	E17	.31801
V11	12.09700		
E01	1.00000	E11	.11988
E17	1.29050	V11	12.14700
E0N7	1.00000	E195	.04684
E1N7	.76887	E1P8	.15355
V11	12.47400		
E0P8	1.00000	E195	.25673
E1P8	.43179	V11	11.99600
E07	1.00000	V11	1.71870
E15	1.00000	E25	.34839
E26	.09371	E298	.19893
E295	.00001	E20	.05026
E21	.00537	E2N7	.00043
E2P8	.00003	E27	1.44436
C12	.35578	C21	.43544
C42	.96500	V12	12.40100
E16	1.00000	E26	.99148
E295	.00017	E2N7	.00749
E2P8	.00079	C12	.00874
C21	.00509	V12	12.45400
E18	1.00000	E28	.99576
E298	.00284	E20	.00059
E21	.00006	E27	.00380
C12	.00434	C21	.00402
C42	.03500	V12	12.55900
E198	1.00000	E298	.42152
E20	.22178	E21	.03117

EXECUTER.	MPS/360	V2-M8	
Y198	1.77141	C12	.54663
Y198	.66566	V12	12.04600
Y198	5.00000		
Y195	1.00000	E295	.20692
Y195	.22178	E21	.03117
Y195	1.77141	C12	.54663
Y195	.66566	V12	12.04600
Y195	5.00000		
Y10	1.00000	E20	.64697
Y10	.13615	E27	.31801
Y10	.44471	C21	.41181
Y10	12.09700	C51	5.00000
Y11	1.00000	E21	.11988
Y11	1.29050	C12	.89365
Y11	1.05392	V12	12.14700
Y11	5.00000		
Y1N7	1.00000	E295	.04684
Y1N7	.76887	E2P8	.15355
Y1N7	.26841	C21	.24855
Y1N7	12.47400	C101	1.00000
Y1P8	1.00000	E295	.25673
Y1P8	.43179	C12	.85765
Y1P8	.79420	V12	11.99600
Y17	5.00000		
Y17	1.00000	C12	.05066
Y17	.04692	V12	1.71870
Y17	1.00000		
Y25	1.00000	E35	.34839
Y25	.09371	E398	.19893
Y25	.00001	E30	.05026
Y25	.00537	E3N7	.00043
Y25	.00003	E37	1.44436
Y25	.35578	C22	.43544
Y25	.56500	V13	12.40100
Y26	1.00000	E36	.94148
Y26	.00017	E3N7	.00749
Y26	.00079	C13	.00874
Y26	.00809	V13	12.45400
Y28	1.00000	E38	.99576
Y28	.00284	E30	.00059
Y28	.00006	E37	.00380
Y28	.00434	C22	.00402
Y28	.03500	V13	12.55900
Y298	1.00000	E398	.42152
Y298	.22178	E31	.03117
Y298	1.77141	C13	.54663
Y298	.66566	V13	12.04600
Y298	5.00000		
Y295	1.00000	E395	.20692
Y295	.22178	E31	.03117
Y295	1.77141	C13	.54663
Y295	.66566	V13	12.04600
Y295	5.00000		
Y20	1.00000	E30	.64697

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Y20	E31	-	.13615	E37	.31801
Y20	C13	-	.44471	C22	.41181
Y21	V13	-	12.09700	C52	5.00000
Y21	E21	-	1.00000	E31	.11988
Y21	E37	-	1.29050	C13	.89365
Y21	C22	-	1.05392	V13	12.14700
Y21	C72	-	5.00000		
Y2N7	E2N7	-	1.00000	E395	.04684
Y2N7	E3N7	-	.76887	E3P8	.15355
Y2N7	C13	-	.26841	C22	.24855
Y2N7	V13	-	12.47400	C102	1.00000
Y2P8	E2P8	-	1.00000	E395	.25673
Y2P8	E3P8	-	.43179	C13	.85765
Y2P8	C22	-	.79420	V13	11.99600
Y2P8	C112	-	5.00000		
Y27	E27	-	1.00000	C13	.05066
Y27	C22	-	.04692	V13	1.71870
Y27	C62	-	1.00000		
Y35	E35	-	1.00000	E45	.34839
Y35	E46	-	.09371	E498	.19893
Y35	E495	-	.00001	E40	.05026
Y35	E41	-	.00537	E4N7	.00043
Y35	E4P8	-	.00003	E47	1.44436
Y35	C14	-	.35578	C23	.43544
Y35	C44	-	.96500	V14	12.40100
Y36	E36	-	1.00000	E46	.99148
Y36	E498	-	.00017	E4N7	.00749
Y36	E4P8	-	.00079	C14	.00874
Y36	C23	-	.00009	V14	12.45400
Y38	E38	-	1.00000	E48	.99576
Y38	E498	-	.00284	E40	.00059
Y38	E41	-	.00006	E47	.00380
Y38	C14	-	.00434	C23	.00402
Y38	C44	-	.03500	V14	12.55900
Y398	E398	-	1.00000	E498	.42152
Y398	E40	-	.22178	E41	.03117
Y398	E47	-	1.77141	C14	.54663
Y398	C23	-	.66566	V14	12.04600
Y398	C03	-	5.00000		
Y395	E395	-	1.00000	E495	.20692
Y395	E40	-	.22178	E41	.03117
Y395	E47	-	1.77141	C14	.54663
Y395	C23	-	.66566	V14	12.04600
Y395	C33	-	5.00000		
Y30	E30	-	1.00000	E40	.64697
Y30	E41	-	.13615	E47	.31801
Y30	C14	-	.44471	C23	.41181
Y30	V14	-	12.09700	C53	5.00000
Y31	E31	-	1.00000	E41	.11988
Y31	E47	-	1.29050	C14	.89365
Y31	C23	-	1.05392	V14	12.14700
Y31	C73	-	5.00000		
Y3N7	E3N7	-	1.00000	E495	.04684
Y3N7	E4N7	-	.76887	E4P8	.15355

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Y3N7	C14	-	.26841	C23
Y3N7	V14	-	12.47400	C103
Y3P8	E3P8	-	1.00000	E495
Y3P8	E4P8	-	.43179	C14
Y3P8	C23	-	.79420	V14
Y3P8	C113	-	5.00000	
Y37	E37	-	1.00000	C14
Y37	C23	-	.04692	V14
Y37	C63	-	1.00000	
Y45	E45	-	1.00000	E55
Y45	E56	-	.09371	E598
Y45	E595	-	.00001	E50
Y45	E51	-	.00537	E5N7
Y45	E5P8	-	.00003	E57
Y45	C15	-	.35578	C24
Y45	C45	-	.96500	V15
Y46	E46	-	1.00000	E56
Y46	E595	-	.00017	E5N7
Y46	E5P8	-	.00079	C15
Y46	C24	-	.00809	V15
Y48	E48	-	1.00000	E58
Y48	E598	-	.00284	E50
Y48	E51	-	.00006	E57
Y48	C15	-	.00434	C24
Y48	C45	-	.03500	V15
Y498	E498	-	1.00000	E598
Y498	E50	-	.22178	E51
Y498	E57	-	1.77141	C15
Y498	C24	-	.66566	V15
Y498	C04	-	5.00000	
Y495	E495	-	1.00000	E595
Y495	E50	-	.22178	E51
Y495	E57	-	1.77141	C15
Y495	C24	-	.66566	V15
Y495	C34	-	5.00000	
Y40	E40	-	1.00000	E50
Y40	E51	-	.13615	E57
Y40	C15	-	.44471	C24
Y40	V15	-	12.09700	C54
Y41	E41	-	1.00000	E51
Y41	E57	-	1.29050	C15
Y41	C24	-	1.05392	V15
Y41	C74	-	5.00000	
Y4N7	E4N7	-	1.00000	E595
Y4N7	E5N7	-	.76887	E5P8
Y4N7	C15	-	.26841	C24
Y4P8	E4P8	-	12.47400	C104
Y4P8	E5P8	-	1.00000	E595
Y4P8	C24	-	.43179	C15
Y4P8	C114	-	.79420	V15
Y47	E47	-	5.00000	
Y47	C24	-	1.00000	C15
Y47	C64	-	.04692	V15
Y47		-	1.00000	
		-	.24855	
		-	1.00000	
		-	.25673	
		-	.85765	
		-	11.99600	
		-	.05066	
		-	1.71870	
		-	.34839	
		-	.19893	
		-	.05026	
		-	.00043	
		-	1.44436	
		-	.43544	
		-	12.40100	
		-	.99148	
		-	.00749	
		-	.00874	
		-	12.45400	
		-	.99576	
		-	.00059	
		-	.00380	
		-	.00402	
		-	12.55900	
		-	.42152	
		-	.03117	
		-	.54663	
		-	12.04600	
		-	.20692	
		-	.03117	
		-	.54663	
		-	12.04600	
		-	.64697	
		-	.31801	
		-	.41181	
		-	5.00000	
		-	.11986	
		-	.89365	
		-	12.14700	
		-	.04684	
		-	.15355	
		-	.24855	
		-	1.00000	
		-	.25673	
		-	.85765	
		-	11.99600	
		-	.05066	
		-	1.71870	

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Y55	OBJ	-	1.00000	E55	1.00000
Y56	C25	-	.43544		
Y58	E56	-	1.00000	C25	.00809
Y598	E58	-	1.00000	C25	.00402
Y595	E598	-	1.00000	C25	.66566
Y50	E595	-	1.00000	C25	.66566
Y51	E50	-	1.00000	C25	.41181
Y5N7	E51	-	1.00000	C25	1.05392
Y5P8	E5N7	-	1.00000	C25	.24855
Y57	E5P8	-	1.00000	C25	.79420
U15	E57	-	1.00000	C25	.04692
U15	OBJ	-	1.00000	E15	1.00000
U16	C21	-	.43544		
U16	E16	-	1.00000	C21	.00809
U18	C91	-	1.00000		
U198	E18	-	1.00000	C21	.00402
U198	COI	-	5.00000	C21	.66566
U195	E198	-	1.00000	C21	.66566
U195	E195	-	5.00000		
U10	C31	-	1.00000	C21	.41181
U10	E10	-	5.00000		
U11	C51	-	1.00000	C21	1.05392
U11	E11	-	5.00000		
U1N7	C71	-	1.00000	C21	.24855
U1P8	E1N7	-	1.00000	C21	.79420
U1P8	E1P8	-	5.00000		
U17	C111	-	1.00000	C21	.04692
U25	E17	-	1.00000	E25	1.00000
U25	OBJ	-	.43544		
U26	C22	-	1.00000	C22	.00809
U26	E26	-	1.00000		
U28	C92	-	1.00000	C22	.00402
U298	E28	-	1.00000	C22	.66566
U298	E298	-	5.00000		
U295	CO2	-	1.00000	C22	.66566
U295	E295	-	5.00000		
U20	C32	-	5.00000	C22	.66566
U20	E20	-	1.00000	C22	.41181
U21	C52	-	5.00000		
U21	E21	-	1.00000	C22	1.05392
U21	C72	-	5.00000		
U2N7	E2N7	-	1.00000	C22	.24855
U2P8	E2P8	-	1.00000	C22	.79420
U2P8	C112	-	5.00000		
U27	E27	-	1.00000	C22	.04692
U35	OBJ	-	1.00000	E35	1.00000
U35	C23	-	.43544		
U36	E36	-	1.00000	C23	.00809
U36	C93	-	1.00000		
U38	E38	-	1.00000	C23	.00402
U398	E398	-	1.00000	C23	.66566
U398	CO3	-	5.00000		
U395	E395	-	1.00000	C23	.66566
U395	C33	-	5.00000		

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EXECUTCR.

U30	E30	-	1.00000	C23	-	.41181
U30	E33	-	5.00000			
U31	E31	-	1.00000	C23	-	1.05392
U31	C73	-	5.00000			
U3N7	E3N7	-	1.00000	C23	-	.24855
U3P8	E3P8	-	1.00000	C23	-	.79420
U3P8	C113	-	5.00000			
U37	E37	-	1.00000	C23	-	.04692
U45	O8J	-	1.00000	E45	-	1.00000
U45	C24	-	.43544			
U46	E46	-	1.00000	C24	-	.00809
U46	C94	-	1.00000			
U48	E48	-	1.00000	C24	-	.00402
U498	E498	-	1.00000	C24	-	.66566
U498	C04	-	5.00000			
U495	E495	-	1.00000	C24	-	.66566
U495	C34	-	5.00000			
U40	E40	-	1.00000	C24	-	.41181
U40	C54	-	5.00000			
U41	E41	-	1.00000	C24	-	1.05392
U41	C74	-	5.00000			
U4N7	E4N7	-	1.00000	C24	-	.24855
U4P8	E4P8	-	1.00000	C24	-	.79420
U4P8	C114	-	5.00000			
U47	E47	-	1.00000	C24	-	.04692
RHS	E05	-	10.53830	E08	-	297.89916
RHS	C12	-	.39941	C13	-	.39941
RHS	C14	-	.39941	C15	-	.39941
RHS	C21	-	.36986	C22	-	.36986
RHS	C23	-	.36986	C24	-	.36986
RHS	C25	-	.36986	V11	-	3900.00000
RHS	V12	-	3900.00000	V13	-	3900.00000
RHS	V14	-	3900.00000	V15	-	3900.00000

ENDATA

EXECUTER. MPS/360 V2-M8

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PICTURE - USING PROFILE

TIME = 0.71

SUMMARY OF MATRIX

SYMBOL	RANGE	COUNT (INCL.RHS)
Z	LESS THAN .000001	
Y	.000001 THRU .000009	5
X	.000010 .000099	10
W	.000100 .000999	20
V	.001000 .009999	46
U	.010000 .099999	42
T	.100000 .999999	175
I	1.000000 1.000000	117
A	1.000001 10.000000	74
B	10.000001 100.000000	46
C	100.000001 1,000.000000	1
D	1,000.000001 10,000.000000	5
E	10,000.000001 100,000.000000	
F	100,000.000001 1,000,000.000000	
G	GREATER THAN 1,000,000.000000	

EXECUTOR. MPS/360 V2-M8

ITER NUMBER	NUMBER INFEAS	VECTOR CLT	VECTOR IN	REDUCED COST	SUM INFEAS
35		13	112	1.53159-	311.984
36	11	51	210	26140.0-	311.984
37		17	173	16.3354-	311.984
38	11	19	177	30.0442-	311.984
39		52	211	.56534-	311.984
40	11	20	179	3518.73-	311.984
41		53	162	7.16282-	311.984
42	11	21	180	44635.1-	311.984
43		57	163	26.6205-	311.984
44	11	59	167	21.5520-	311.984
45		23	172	.51585-	311.984
46		192	201	.44835-	311.984
47	11	60	169	18136.5-	311.984
48		25	183	5.52955-	311.984
49		26	135	4.17006-	311.984
50		27	136	142460.-	311.984
51	11	61	170	230065.-	311.984
52		29	187	14.9384-	311.984
53		172	181	.90611-	311.984
54		8	118	.13448-	311.984
55		44	204	.00402-	311.984
56		24	184	.00402-	311.984
57	11	30	189	10503.1-	311.984
58		14	114	1.00307-	311.984
59		5	115	.37986-	311.984
60		6	116	.09412-	311.984
61		34	194	.00402-	311.984
62	11	31	190	133234.-	311.984
63	11	81	141	2.20200-	311.984
64	11	182	185	26.4284-	311.984
65	11	211	198	2.19795-	311.984
66		148	171	47.2554-	311.984
67	11	85	161	15.4096-	311.984

INVERT DEMANDED AFTER 16 MAJOR/ 40 MINOR ITERATIONS - CLOCK CONTROL

INVERT CALLED	TIME	1.96	CURRENT INVERSE	-----	ETA-VECTORS	92
BASIS -----	NO.OF ROWS	...111	LOGICALS	49	STRUCTURALS	62
INVERSE --	NUCLEUS	22	TRANSFORMED	4	ETA-VECTORS	93
					ELEMENTS	...2350
					ELEMENTS	...345
					ELEMENTS	...344
					RECORDS	2
					RECORDS	1
					TIME TAKEN	0.04
					ITERATION	57

PRIMAL OBJ = OBJ RHS = RHS

TIME = 2.02 MINS. PRICING 7
SCALE = .

ITER NUMBER	NUMBER INFEAS	VECTOR CLT	VECTOR IN	REDUCED COST	SUM INFEAS
68	11	93	191	.05066-	311.984
69	11	95	211	.05066-	311.984
70	11	91	192	.82076-	311.984
71		142	182	.28006-	311.984

EXECUTOR. MPS/360 V2-M8

ITER NUMBER	NUMBER INFEAS	VECTOR CUT	VECTOR IN	REDUCED COST	SUM INFEAS
M 72	11	74	202	.55621-	311.984
M 73	11	54	154	.01737-	311.984
M 74	9	79	164	.01343-	311.215
M 75	9	58	148	.95944-	311.215
M 76	9	50	157	56.8445-	311.215
M 77	9	182	186	.10E+07-	311.215
M 78	9	103	142	.55621-	311.215
M 79	9	111	153	5.93516-	311.215
M 80	9	88	147	172.287-	311.215
M 81	9	102	182	.61234-	311.215
M 82	9	89	172	.63930-	311.215
M 83	122	127	127	31.5335-	311.215

INVERT DEMANDED AFTER 13 MAJOR/ 16 MINOR ITERATIONS - CLOCK CONTROL

INVERT CALLED	TIME 2.34	CURRENT INVERSE	ETA-VECTORS	109
BASIS ----	NO.OF ROWS	LOGICALS	STRUCTURALS	75
INVERSE --	NUCLEUS	TRANSFORMED	ETA-VECTORS	116
			RECORDS	988
			ELEMENTS	388
			ELEMENTS	474
			RECORDS	1
			ITERATION	25
			TIME TAKEN	0.64

PRIMAL OBJ = C8J RHS = RHS

TIME = 2.40 MINS. PRICING 7
SCALE =

ITER NUMBER	NUMBER INFEAS	VECTOR CUT	VECTOR IN	REDUCED COST	SUM INFEAS
M 84	9	172	137	66.0952-	311.215
M 85	182	122	122	4.25644-	311.215
M 86	5	211	143	123.933-	311.215
M 87	192	160	160	397744-	311.215
M 88	9	97	211	13.9014-	311.215
M 89	9	110	182	.80433-	311.215
M 90	9	73	192	.55621-	311.215
M 91	7	204	144	.01337-	310.445
M 92	7	71	188	338.779-	310.445
M 93	7	184	124	1.15044-	310.445
M 94	7	182	134	1.14854-	310.445
M 95	7	100	150	.54E+07-	310.445
M 96	7	72	182	.55621-	310.445

INVERT DEMANDED AFTER 11 MAJOR/ 13 MINOR ITERATIONS - CLOCK CONTROL

INVERT CALLED	TIME 2.69	CURRENT INVERSE	ETA-VECTORS	129
BASIS ----	NO.OF ROWS	LOGICALS	STRUCTURALS	81
INVERSE --	NUCLEUS	TRANSFORMED	ETA-VECTORS	131
			RECORDS	1422
			ELEMENTS	452
			ELEMENTS	600
			RECORDS	1
			ITERATION	75
			TIME TAKEN	0.21

PRIMAL OBJ = C8J RHS = RHS

TIME = 2.91 MINS. PRICING 7
SCALE =

EXECUTOR. MPS/360 V2-M8

ITER NUMBER	NUMBER INFEAS	VECTOR CUT	VECTOR IN	REDUCED COST	SUM INFEAS
M 97	4	4	71	28.8589-	14.2815
M 98	4	101	172	.55621-	14.2815
M 99		122	123	7.50473-	14.2815
M 100	4	108	133	13.5044-	14.2815
M 101	4	109	122	.55621-	14.2815
M 102	2	78	140	32.882.6-	11.7424
M 103	2	67	130	70.968.4-	10.0580
M 104	0	2	72	10.9007-	.

FEASIBLE SOLUTION

PRIMAL OBJ = CBJ RHS = RHS

TIME = 3.15 MINS. PRICING 7

SCALE =
SCALE RESET TO 1.00000

ITER NUMBER	NUMBER NCNOPT	VECTOR CUT	VECTOR IN	REDUCED COST	FUNCTION VALUE
M 105	11	68	74	8.66896-	16.3913
M 106	4	69	73	1.16007-	9.91488
M 107		70	79	.05254-	9.39484
M 108	2	194	78	.00119-	9.27799

OPTIMAL SOLUTION

EXECUTOR. MPS/360 VZ-M8

SOLUTION (OPTIMAL)

TIME = 3.31 MINS. ITERATION NUMBER = 108

...NAME...	...ACTIVITY...	DEFINED AS
FUNCTIONAL	9.27799	OBJ
RESTRAINTS		RHS

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EXECUTOR. MPS/360 V2-M8

SECTION 1 - ROWS

NUMBER	...ROW...	AT	...ACTIVITY...	SLACK ACTIVITY	...LOWER LIMIT...	...UPPER LIMIT...	DUAL ACTIVITY
1	CEJ	BS	9.27799	9.27799-	NONE	NONE	1.00000
2	E05	EC	10.63830	.	10.63830	10.63830	.56583
3	EC6	BS	.	.	.	.	.
4	EC8	EC	297.89916	.	297.89916	297.89916	.02557-
5	EC58	EC	.	.	.	.	.03564
6	EC55	EC	.	.	.	.	.07669-
7	EC0	BS	.	.	.	.	.
8	EC1	EC	.	.	.	.	.01314
9	ECN7	BS	.	.	.	.	.
10	ECP8	BS	.	.	.	.	.
11	E07	BS	.	.	.	.	.
12	E15	EC	.	.	.	.	1.00000-
13	E16	EC	.	.	.	.	.12993
14	E18	EC	.	.	.	.	.02930
15	E196	EC	.	.	.	.	.23652-
16	E195	EC	.	.	.	.	.04125
17	E10	EC	.	.	.	.	.30466
18	E11	EC	.	.	.	.	.10958-
19	E1N7	EC	.	.	.	.	.
20	E1P8	EC	.	.	.	.	.34826
21	E17	EC	.	.	.	.	.
22	E25	EC	.	.	.	.	2.36413-
23	E26	EC	.	.	.	.	.12326
24	E28	EC	.	.	.	.	.03473
25	E293	EC	.	.	.	.	2.32254-
26	E295	EC	.	.	.	.	2.04705-
27	E20	EC	.	.	.	.	.56173
28	E21	EC	.	.	.	.	3.42912-
29	E2N7	EC	.	.	.	.	.77950
30	E2P8	EC	.	.	.	.	2.63015
31	E27	EC	.	.	.	.	.14098
32	E35	EC	.	.	.	.	2.35468-
33	E36	EC	.	.	.	.	.09104
34	E38	EC	.	.	.	.	.02759
35	E398	EC	.	.	.	.	2.31048-
36	E395	EC	.	.	.	.	2.04109-
37	E30	EC	.	.	.	.	1.55975
38	E31	EC	.	.	.	.	2.35704-
39	E3N7	EC	.	.	.	.	.77325
40	E3P8	EC	.	.	.	.	2.79217
41	E37	EC	.	.	.	.	.14376
42	E45	EC	.	.	.	.	.34811-
43	E46	EC	.	.	.	.	.02904
44	E48	EC	.	.	.	.	.03044
45	E498	EC	.	.	.	.	2.2573-
46	E495	EC	.	.	.	.	1.05455-
47	F40	EC	.	.	.	.	1.49302
48	E41	EC	.	.	.	.	3.36982-
49	E4N7	EC	.	.	.	.	.76991

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NUMBER	ROW	AT	ACTIVITY	SLACK	ACTIVITY	LOWER LIMIT	UPPER LIMIT	QUAL	ACTIVITY
50	E4P8	EC	.	.	.	.	.	.	4.73845
51	E47	EC	.	.	.	.	.	.	.14525
52	E55	EC	.	.	.	.	.	.	2.41503-
53	E56	EC	.	.	.	.	.	.	.62650
54	F58	EC	.	.	.	.	.	.	.01305
55	E598	EC	.	.	.	.	.	.	2.16406-
56	E595	EC	.	.	.	.	.	.	2.16405-
57	F50	EC	.	.	.	.	.	.	1.33279
58	E51	EC	.	.	.	.	.	.	3.42630-
59	F5N7	EC	.	.	.	.	.	.	.80203
60	E5P8	EC	.	.	.	.	.	.	2.58195
61	E57	EC	.	.	.	.	.	.	.15253
62	C12	RS	1.08010	.	68069-	35941	.	NONE	.
63	C13	BS	1.00159	.	60218-	35941	.	NONE	.
64	C14	BS	.99628	.	59387-	35941	.	NONE	.
65	C15	BS	1.00159	.	60218-	35941	.	NONE	.
66	C21	HS	1.39797	.	1.02811-	36986	.	NONE	.
67	C22	LL	.36986	.	.	36986	.	NONE	3.13275-
68	C23	LL	.36986	.	.	36986	.	NONE	3.11104-
69	C24	LL	.36986	.	.	36986	.	NONE	3.09995-
70	C25	LL	.36986	.	.	36986	.	NONE	3.25104-
71	C42	BS	5.79330-	.	5.79380	NONE	.	.	.
72	C43	BS	5.48724-	.	5.48724	NONE	.	.	.
73	C44	BS	5.40367-	.	5.40367	NONE	.	.	.
74	C45	BS	5.34717-	.	5.34717	NONE	.	.	.
75	V11	BS	3873.24111	.	26.75889	NONE	3900.00000	.	.
76	V12	BS	3819.38132	.	80.61668	NONE	3900.00000	.	.
77	V13	HS	3811.15472	.	88.84528	NONE	3900.00000	.	.
78	V14	RS	3801.59192	.	98.40508	NONE	3900.00000	.	.
79	V15	BS	3791.82412	.	108.17588	NONE	3900.00000	.	.
80	C01	EC	.	.	.	.	.	.	.04730
81	C02	EC	.	.	.	.	.	.	.04750
82	C03	EC	.	.	.	.	.	.	.04792
83	C04	EC	.	.	.	.	.	.	.04519
84	C31	EC	.	.	.	.	.	.	.00825-
85	C32	EC	.	.	.	.	.	.	.00769-
86	C33	EC	.	.	.	.	.	.	.00516-
87	C34	EC	.	.	.	.	.	.	.00125-
88	C51	EC	.	.	.	.	.	.	.00073-
89	C52	EC	.	.	.	.	.	.	.03144-
90	C53	EC	.	.	.	.	.	.	.05572-
91	C54	EC	.	.	.	.	.	.	.04432-
92	C61	RS	.	.	.	.	.	.	.
93	C62	EC	.	.	.	.	.	.	.
94	C63	BS	.	.	.	.	.	.	.
95	C64	EC	.	.	.	.	.	.	.
96	C71	EC	.	.	.	.	.	.	.02192
97	C72	EC	.	.	.	.	.	.	.02177
98	C73	EC	.	.	.	.	.	.	.01135
99	C74	EC	.	.	.	.	.	.	.02179
100	C91	EC	.	.	.	.	.	.	.12992

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NUMBER	...ROW...	AT	...ACTIVITY...	SLACK	ACTIVITY	..LOWER LIMIT.	..UPPER LIMIT.	..DUAL ACTIVITY
101	C92	EC	.	.	.	.	.	.09752
102	C93	EC	.	.	.	.	.	.06558
103	C94	EC	.	.	.	.	.	.03360
104	C101	BS	.	.	.	.	.	.
105	C102	BS	.	.	.	.	.	.
106	C103	BS	.	.	.	.	.	.
107	C104	BS	.	.	.	.	.	.
108	C111	EC	.	.	.	.	.	.06365-
109	C112	EC	.	.	.	.	.	.06343-
110	C113	EC	.	.	.	.	.	.06550-
111	C114	EC	.	.	.	.	.	.05593-

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SECTION 2 - COLUMNS

NUMBER	COLUMN	AT	ACTIVITY...	INPUT COST..	LOWER LIMIT.	UPPER LIMIT.	REDUCED COST.
112	Y05	BS	10.63030	.	.	NONE	.
113	Y06	LL	.	.	.	NONE	.12911
114	Y08	BS	297.85916	.	.	NONE	.
115	Y05B	BS	.	.	.	NONE	.
116	Y055	BS	.	.	.	NONE	.
117	Y00	LL	.	.	.	NONE	.10219
118	Y01	BS	.	.	.	NONE	.
119	Y0N7	LL	.	.	.	NONE	.05541
120	Y0P8	LL	.	.	.	NONE	.16096
121	Y07	LL	.	.	.	NONE	.
122	Y15	BS	4.74974	.	.	NONE	.
123	Y16	BS	.54696	.	.	NONE	.
124	Y18	BS	296.63705	.	.	NONE	.
125	Y158	BS	1.48159	.	.	NONE	.
126	Y155	BS	.00003	.	.	NONE	.
127	Y10	BS	.35514	.	.	NONE	.
128	Y11	BS	.03730	.	.	NONE	.
129	Y1N7	LL	.	.	.	NONE	.93735
130	Y1P8	BS	.00018	.	.	NONE	.
131	Y17	LL	.	.	.	NONE	.
132	Y25	BS	5.02701	.	.	NONE	.
133	Y26	BS	1.43359	.	.	NONE	.
134	Y28	BS	295.38029	.	.	NONE	.
135	Y258	BS	1.20635	.	.	NONE	.
136	Y295	BS	.00012	.	.	NONE	.
137	Y20	BS	.48595	.	.	NONE	.
138	Y21	BS	.07095	.	.	NONE	.
139	Y2N7	LL	.	.	.	NONE	.92561
140	Y2P8	BS	.00051	.	.	NONE	.
141	Y27	BS	.	.	.	NONE	.
142	Y35	BS	5.06823	.	.	NONE	.
143	Y36	BS	1.89248	.	.	NONE	.
144	Y36	BS	294.12686	.	.	NONE	.
145	Y358	BS	1.17414	.	.	NONE	.
146	Y355	BS	.00021	.	.	NONE	.
147	Y30	BS	.50436	.	.	NONE	.
148	Y31	BS	.07829	.	.	NONE	.
149	Y3N7	LL	.	.	.	NONE	.91591
150	Y3P8	BS	.00076	.	.	NONE	.
151	Y37	LL	.	.	.	NONE	.
152	Y45	BS	5.08158	.	.	NONE	.
153	Y46	BS	2.35133	.	.	NONE	.
154	Y46	BS	292.88273	.	.	NONE	.
155	Y452	BS	1.16452	.	.	NONE	.
156	Y495	BS	.00013	.	.	NONE	.
157	Y40	BS	.50741	.	.	NONE	.
158	Y41	BS	.07956	.	.	NONE	.
159	Y4N7	LL	.	.	.	NONE	.91677
160	Y4P8	BS	.00100	.	.	NONE	.

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NUMBER	COLUMN.	AT	ACTIVITY...	INPUT COST..	LOWER LIMIT.	UPPER LIMIT.	REDUCED COST.
161	Y47	BS	1.77039	1.00000-	.	NONE	.
162	Y55	BS	2.80752	.	.	NONE	.
163	Y56	BS	291.64187	.	.	NONE	.
164	Y58	BS	2.33663	.	.	NONE	.
165	Y598	BS	.CC07C	.	.	NONE	.
166	Y595	BS	1.01577	.	.	NONE	.
167	Y50	BS	.15955	.	.	NONE	.
168	Y51	BS	.01979	.	.	NONE	.
169	Y5N7	BS	.CC245	.	.	NONE	.
170	Y5P8	BS	10.78932	1.00000	.	NONE	.
171	Y57	BS	1.04343	.	.	NONE	.
172	U15	BS	.	.	.	NONE	.
173	U16	BS	.	.	.	NONE	.
174	U18	LL	1.48159	.	.	NONE	.02930
175	U198	BS	.CCCC3	.	.	NONE	.
176	U195	BS	.25514	.	.	NONE	.
177	U10	BS	.03730	.	.	NONE	.
178	U11	BS	.CC457	.	.	NONE	.
179	U1N7	BS	.CC018	.	.	NONE	.
180	U1P8	BS	16.49779	1.00000	.	NONE	.
181	U17	BS	3.37224	.	.	NONE	.
182	U25	BS	.	.	.	NONE	.
183	U26	BS	.	.	.	NONE	.02213
184	U28	LL	1.20635	.	.	NONE	.
185	U298	BS	.CC012	.	.	NONE	.
186	U295	BS	.48595	.	.	NONE	.
187	U20	BS	.07095	.	.	NONE	.
188	U21	BS	.CC950	.	.	NONE	.
189	U2N7	BS	.00051	.	.	NONE	.
190	U2P8	BS	10.77340	1.00000	.	NONE	.
191	U27	BS	3.31686	.	.	NONE	.
192	U35	BS	.	.	.	NONE	.01509
193	U36	BS	1.17414	.	.	NONE	.
194	U38	LL	.CC021	.	.	NONE	.
195	U358	BS	.50436	.	.	NONE	.
196	U395	BS	.07829	.	.	NONE	.
197	U30	BS	.C1289	.	.	NONE	.
198	U31	BS	.CC076	.	.	NONE	.
199	U3N7	BS	10.76675	1.00000	.	NONE	.
200	U3P8	BS	3.31585	.	.	NONE	.
201	U37	BS	.	.	.	NONE	.
202	U45	BS	1.16982	.	.	NONE	.00901
203	U46	BS	.CC013	.	.	NONE	.
204	U48	LL	.50741	.	.	NONE	.
205	U498	BS	.07556	.	.	NONE	.
206	U495	BS	.C1035	.	.	NONE	.
207	U40	BS	.CC100	.	.	NONE	.
208	U41	BS	10.77994	.	.	NONE	.
209	U4N7	BS	.	.	.	NONE	.
210	U4P8	BS	.	.	.	NONE	.
211	U47	BS	.	.	.	NONE	.

A LINEAR PROGRAMMING REACTOR

REFUELING MODEL

by

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AN ABSTRACT OF A MASTER'S THESIS

submitted in partial fulfillment of the

requirements for the degree

MASTER OF SCIENCE

Department of Nuclear Engineering

KANSAS STATE UNIVERSITY
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1971

ABSTRACT

A linear programming reactor refueling model was developed to minimize the U-235 requirements of a power reactor. The model consisted of a fuel cycle of five, one year, irradiation periods.

A fuel burnup code was used which made the following assumptions: 1) one homogeneous region, 2) known neutron fluence for each period, 3) one neutron energy group (thermal), 4) no U-238 fission, and 5) batch irradiation. The burnup equations were linearized for the model by using only nuclide concentrations at the end points of each irradiation period.

The constraints of the model include a maximum and minimum enrichment constraint, a multiplication factor constraint, a volume constraint, and nuclide refueling and removal constraints.

Several different refueling cases with varying degrees of Pu recycle were studied. The effect of the percent of Pu recycle on the burnup and buildup of the nuclides in the reactor was also studied. This study revealed that the value of plutonium decreases as the percent of Pu recycled increases.